PROPOSED AMENDMENT TO

TECHNICAL SPECIFICATIONS

for

DRESDEN STATION UNITS 2 and 3

and

QUAD CITIES STATION UNITS 1 and 2

Concerning EGC Operation

DPR-19 & DPR-25	Proposed Page Changes
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DPR-29 & DPR-30	3.3/4.3-5

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3.3 LIMITING CONDITION FOR OPERATION	4.3 SURVEILLANCE REQUIREMENT
E. Reactivity Anomalies The reactivity equivalent of the difference between the actual critical rod configuration and the expected configuration during power operation shall not exceed 1% ΔK. If this limit is exceeded, the reactor will be shutdown until the cause has been determined and corrective actions have been taken if such actions are appropriate. In accordance with Specification 6.6, the NRC shall be notified of this reportable occurrence within 24 hours.	E. Reactivity Anomalies During the startup test program and startups following refueling outages, the critical rod configurations will be compared to the expected configurations at selected operating conditions. These comparisons will be used as base data for reactivity monitoring during subsequent power operation throughout the fuel cycle. At specific power operating conditions, the critical rod configuration will be compared to the configuration expected based upon appropriately corrected past data. This comparison will be made at least every equivalent full power month.
F. If Specifications 3.3.A through D above are not met, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown condition within 24 hours.	
G. Economic Generation Control System Operation of the unit with the Economic Generation Control system with automatic flow control shall be permissible only in the range of 65-100% of rated core flow, with reactor power above 20%.	G. Economic Generation Control System Prior to entering EGC and once per shift while operating in EGC, the EGC operating parameters will be reviewed for acceptability.

Table 6.6.1
SPECIAL REPORTS

	AREA	SPECIFICATION REFERENCE	SUBMITTAL DATE
a.	Response time of safety related instruments (2)	1.0.E (Dres. 1)	Annual Report
b.	Main stream isolation valve and feedwater power operated isolation valves closure times (2)	3.7.B.1.c (Dres. 1)	Annual Report
c.	Primary Coolant leakage to Drywell (4)	4.6.D Bases	5 years (1)/
d.	In-Service Inspection Evaluation (4)	Table 4.6.1	5 years (1)
e.	Evaluation of EGCS operation (4)	3.3.G Bases	Upon completion of initial testing
f.	Failed Fuel Detection (4)	3.2 Bases	5 years (1)
g.	Main Steam Line Leakage to Steam Tunnel (4)	4.6.D Bases	5 years (1)
h.	In-service Inspection Development (4)	4.6.1 Bases	5 years (1)
i.	In-Service Inspection of Sensitized Stainless Steel Components (3)	4.6.F	4 years (1)
j.	Secondary Containment Leak Rate Test (4)	3.7.C.1	within 90 days after complotion of each test
k.	High off-gas discharge rate (2)	3.8.A.4 (Dresden 1)	within 24 hours of occurrence
l.	Radioactive Source Leak Testing (5)	3.8.F	Annual Report

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k.	High off-gas discharge rate (2)	3.8.A.4 (Dresden 1)	within 24 hours of occurrence
l.	Radioactive Source Leak Testing (5)	3.0.F	Annual Report

sidered inoperable, fully inserted into the core, and electrically disarmed.

5. If the overall average of the 20% insertion scram time data generated to date in the current cycle exceeds 0.73 seconds, the MCPR operating limit must be modified as required by Specification 3.5.K.

provide reasonable assurance that proper control rod drive performance is being maintained. The results of measurements performed on the control rod drives shall be submitted in the annual operating report to the NRC.

3. The cycle cumulative mean scram time for 20% insertion will be determined immediately following the testing required in Specifications 4.3.C.1 and 4.3.C.2 and the MCPR operating limit adjusted, if necessary, as required by Specification 3.5.K.

D. Control Rod Accumulators

At all reactor operating pressures, a rod accumulator may be inoperable provided that no other control rod in the nine-rod square array around this rod has:

1. an inoperable accumulator,
2. a directional control valve electrically disarmed while in a nonfully inserted position, or
3. a scram insertion greater than maximum permissible insertion time

If a control rod with an inoperable accumulator is inserted full-in and its directional control valves are electrically disarmed, it shall not be considered to have an inoperable accumulator, and the rod block associated with that inoperable accumulator may be bypassed

D. Control Rod Accumulators

Once a shift, check the status of the pressure and level alarms for each accumulator

E. Reactivity Anomalies

The reactivity equivalent of the difference between the actual critical rod configuration and the expected configuration during power operation shall not exceed 1%Δk. If this limit is exceeded, the reactor shall be shutdown until the cause has been determined and corrective actions have been taken. In accordance with Specification 6.6, the NRC shall be notified of this reportable occurrence within 24 hours.

E. Reactivity Anomalies

During the startup test program and startups following refueling outages, the critical rod configurations will be compared to the expected configurations at selected operating conditions. These comparisons will be used as base data for reactivity monitoring during subsequent power operation throughout the fuel cycle. At specific power operating conditions, the critical rod configuration will be compared to the configuration expected based upon appropriately corrected past data. This comparison will be made at least every equivalent full power month

F. Economic Generation Control System

Operation of the unit with the economic generation control system with automatic flow control shall be permissible only in the range of 65% to 100% of rated core flow, with reactor power above 20%.

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sidered inoperable, fully inserted into the core, and electrically disarmed.	provide reasonable assurance that proper control rod drive performance is being maintained. The results of measurements performed on the control rod drives shall be submitted in the annual operating report to the NRC.
5. If the overall average of the 20% insertion scram time data generated to date in the current cycle exceeds 0.73 seconds, the MCPR operating limit must be modified as required by Specification 3.5.K.	5. The cycle cumulative mean scram time for 20% insertion will be determined immediately following the testing required in Specifications 4.3.C.1 and 4.3.C.2 and the MCPR operating limit adjusted, if necessary, as required by Specification 3.5.K.
D. Control Rod Accumulators	**D. Control Rod Accumulators**
At all reactor operating pressures, a rod accumulator may be inoperable provided that no other control rod in the nine-rod square array around this rod has: 1. an inoperable accumulator, 2. a directional control valve electrically disarmed while in a nonfully inserted position, or 3. a scram insertion greater than maximum permissible insertion time. If a control rod with an inoperable accumulator is inserted full-in and its directional control valves are electrically disarmed, it shall not be considered to have an inoperable accumulator, and the rod block associated with that inoperable accumulator may be bypassed.	Once a shift, check the status of the pressure and level alarms for each accumulator.
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