April 23, 1982

**Docket No. 50-237** LS05-82-04-067

> Mr. L. DelGeorge Director of Nuclear Licensing Commonwealth Edison Company Post Office Box 767 Chicago, Illinois 60690

Dear Mr. DelGeorge:

SUBJECT: SEP TOPIC VI-7-A.4 - CORE SPRAY NOZZLE EFFECTIVENESS -

By letter dated February 22, 1982 we provided our evaluation of SEP Topic VI-7.A.4 for the Dresden 2 facility. Our evaluation stated that we had relied upon a General Electric Company analysis that had not been submitted for our review. The analysis still has not been provided for our use.

Please provide the information requested in Enclosure 1 within 45 days of your receipt of this letter so that we can confirm our earlier position that core spray distribution is not a safety concern at Dresden 2.

This request involves fewer than ten respondents. Therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

Original signed by

Dennis M. Crutchfield, Chief Operating Reactors Branch #5 Division of Licensing

Enclosure: Information request

cc w/enclosure: See next page

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cc Isham, Lincoln & Beale Counselors at Law One First National Plaza, 42nd Floor Chicago, Illinois 60603

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Morris, Illinois 60450

The Honorable Tom Corcoran
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U. S. Environmental Protection Agency Federal Activities Branch Region V Office ATTN: Regional Radiation Representative 230 South Dearborn Street Chicago, Illinois 60604

James G. Keppler, Regional Administrator Nuclear Regulatory Commission, Region III 799 Roosevelt Street Glen Ellyn, Illinois 60137

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## DRESDEN 2 INFORMATION REQUEST FOR CORE SPRAY DISTRIBUTION

Information derived from the Japanese core spray tests suggested that the central fuel bundles of a BWR/3 core may receive icw core spray flow.

Dresden-2 and Millstone-1 are two BWR/3 plants which are being reviewed as part of the Systematic Evaluation Frogram (SEP). To respond to the concerns of SEP generic issue A-16, steam effect on BWR core spray distribution, the implication of the core spray test information from Japan on the core spray distribution adequacy for Dresden-2 and Mill-stone-1 should be addressed.

In a telephone conference with GE, the staff was told that the analyses can be done to demonstrate that for a limiting BWR/3 case (Dresden-2 and Millstone-1 are both BWR/3 cores), the calculated peak cladding temperature will not exceed the 10 CFR 50.46 limit of 2200°F even when no credit is taken in the analyses for core spray cooling.

The applicants are requested to provide the results of the analyses including a description of the assumptions used in the analyses. These should be forwarded to NRC within 45 days after receipt of this request.

If the staff's evaluation of the applicants' analyses reveals results which confirm the staff's understanding gained in the telephone conference, we will be able to conclude that the staff's concern regarding generic issue A-16 for Dresden-2 and Millstone-1 has been satisfactorily answered.