

Guides and the requirements of the following ANSI Standards as listed for each station.

I. Dresden, Quad Cities and Zion Nuclear Power Stations

1.28 (Safety Guide 28 - 6/72); 1.30 (Safety Guide 30 - 8/72); 1.33 (Safety Guide 33 - 11/72); 1.37 - 3/73; 1.38 - 3/73; 1.39 - 3/73; 1.54 - 6/73; 1.58 - Rev. 1; 1.64 - 10/73; 1.74 - 2/74; 1.8 (Safety Guide 8 - 3/71); 1.146 - Rev. 0 8/80; ANSI N45.2.8-74 (Draft 3, Rev. 3, 4/74); ANSI N45.2.5-73 (Draft 3, Rev. 1, 1/74); ANSI N45.2.9-74 (Draft 15, Rev. 0, 4/74); ANSI N45.2.12-74 (Draft 3, Rev. 4 2/74); ANSI N45.2.13-74 (Draft 2, Rev. 4, 4/74)

II. LaSalle County Nuclear Power Station

1.28 - Rev. 0, 6/72; 1.30 - Rev. 0, 8/72; 1.33 - Rev. 2; 1.37 - Rev. 0, 3/73; 1.38 - Rev. 2; 1.39 - Rev. 2; 1.54 - 6/73; 1.58 - Rev. 1; 1.64 - Rev. 2; 1.74 - Rev. 0, 2/74; 1.8 - Rev. 1-R; 1.88 - Rev. 2; 1.94 - Rev. 1; 1.116 - Rev. 0-R, 6/76; 1.123 - Rev. 1; 1.146 - Rev. 0, 8/80

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III. Byron and Braidwood Nuclear Power Stations

1.28 - Rev.1; 1.30 - Rev. 0, 8/72; 1.33 - Rev. 2; 1.37 - Rev. 0, 3/73; 1.38 - Rev. 2; 1.39 - Rev. 2; 1.54 - 6/73; 1.58 - Rev. 1; 1.64 - Rev. 2; 1.74 - Rev. 0, 2/74; 1.8 - Rev. 1-R; 1.88 - Rev. 2; 1.94 - Rev. 1; 1.116 - 0-R, 6/76; 1.123 - Rev. 1; 1.146 Rev. 0, 8/80

Exceptions or alternatives to this Topical Report for specific plants identified in the Safety Analysis Report or Technical Specification will take precedence over commitments in this Topical Report.

It is also the policy of Commonwealth Edison Company to assure a high degree of functional integrity for its generating facilities so as to achieve high availability of these facilities for the production of electrical power and to maintain overall quality levels which will achieve the foregoing in a safe, effective and economic manner.

The Quality Requirements and Quality Procedures of the Company Quality Assurance Program Manual (see Appendix A for Program Manual Index) as described herein, document the

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The training and certification of personnel associated with nondestructive examination will be carried out in accordance with the requirements of SNT-TC-1A and the Code. A Level III certified person will administer this activity and the program will be under the surveillance of the Directors of Quality Assurance or their delegated representatives. Likewise, a Level III certified person will administer the concrete containment inspection as applicable under Division 2 of the ASME Code. Training and certification of personnel associated with concrete containment inspection will be carried out in accordance with Appendix VII of ASME Section III, Division 2.

The period of qualification shall be a minimum of two and a maximum of three years. Near the end of such a period, employees will be notified that they will require either retraining and requalification and/or recertification to renew their qualification status.

Contractor personnel engaged in inspection, examination and testing activities will be required to be trained, qualified and certified to perform their specific activity in accordance with the above requirements.

Such training will be documented and the record of those receiving training shall include as a minimum the date, subject and the name of the person who performed the training.

The objectives of the Commonwealth Edison Company Quality Assurance Program are to: (1) establish with confidence that the design, fabrication, erection and operation phases of its nuclear power generation facilities are performed in a manner consistent with the policies stated above; (2) provide for documenting, retaining documentation and updating design, fabrication, erection and product quality information necessary for Commonwealth Edison Company operation, maintenance, repair, modification, refueling and in-service inspection of the nuclear power facilities; (3) avoid schedule delays and high cost due to poor quality; and (4) achieve high plant reliability and availability.

The Quality Assurance Program applies to safety-related and ASME Section III activities and items and related consumables plus fire protection, security, emergency plan, meteorology, radwaste shipment and review of environmental qualification of Class 1E Equipment. The classification of structures, systems and components designated as "safety-related" are listed in the respective Safety Analysis Reports (SAR) of the Stations. A copy of the SAR or the listing will

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