



**Commonwealth Edison**  
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March 13, 1981

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Dennis M. Crutchfield, Chief  
 Operating Reactors Branch #5  
 Division of Licensing  
 U.S. Nuclear Regulatory Commission  
 Washington, DC 20555

Subject: Dresden 2  
 SEP Topic III-6, Seismic Design Considerations  
 and III-11, Component Integrity  
 NRC Docket No: 50-237

Reference (a): Dennis M. Crutchfield's letter of January 30,  
 1981 to D. L. Peoples

Dear Mr. Crutchfield:

In response to Reference (a) Commonwealth Edison Co. will review report EGG-EA-5065, "Summary of the Dresden Unit 2 Piping Calculation" and submit our comments by April 24, 1981.

Edison's responses to enclosure 2 of Reference (a), which have been discussed with the staff during a March 11, 1981 conference call are as follows:

1. Junction of Reactor Building and Turbine Building

The model Blume used of the junction of the reactor and turbine building, along with the design parameters used, the model, and the results of the analysis, will be submitted by April 10, 1981.

2. Control Rod Drive Units and Associated Hydraulic Tubing Supports

The NRC currently has this item under review. No additional information is required of Edison at this time.

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3. Motor-Operated Valves

This item is being addressed by the effort Edison is performing in resolving the NRC Bulletin 79-14. In our April 24 submittal we will provide a brief summary of our results to date. Analysis of the piping systems is currently scheduled to be complete by the end of 1981. Support design for modifications will proceed into 1982.

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4. Reactor Vessel and Internal Shroud Support

Edison provided on February 29, 1980 the information requested during a November 28, 1979 conference call between GE, NRC and Edison. Based on the March 11, 1981 conference call the NRC Staff will arrange for Dr. Stevenson to clarify what additional information is required.

5. Recirculation Pump and Supports

The staff will provide Edison with the consultants report for our review and comment. If the staff feels additional analysis is required of Edison the staff will state what is required in the transmittal letter of the report.

6. Piping

This item is being addressed by the effort Edison is performing in resolving the NRC Bulletin 79-14. In our April 24 submittal we will provide a summary of the piping analysis methods being used to assure the plant meets appropriate stress criteria.

7. Battery Racks

Edison will provide a design to replace or reinforce the wooden lateral bracing and simple hand calculations to support the design by April 24, 1981. Edison will implement the design after the NRC Staff has reviewed and approved the design.

8. Instrumentation and Control Room Panels

The staff will review the March 27, 1980 submittal with their consultants. After the review is complete the staff will inform Edison of what, if any, additional information is required.

9. AC and DC Motor Control Centers

Edison has walked down all of the safety related electrical equipment to determine if it had positive anchorage and to determine the as built configuration of the anchorage. The engineering review and the modification designs on the anchorage systems has been completed. The modifications are currently being

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installed and will be completed during the present outage. The detailed report on what was found and what was modified will be submitted prior to restart of the unit.

Determining the resonance frequencies of the equipment is part of the generic functionality program being developed by Westinghouse for the SEP Owner's Group. The seismic functionality qualification of this equipment will be addressed by the Westinghouse program.

10. Switchgear

This equipment has adequate positive anchorage. The seismic functionality qualification of this equipment will be addressed by the Westinghouse program.

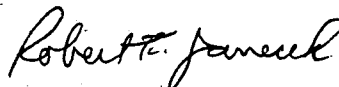
11. Cable Tray

This equipment is being addressed by a generic study being performed for the SEP Owner's Group by URS/J.A. Blume. The initial presentation of the program was given to the staff during a March 5, 1981 meeting.

Please address any questions you may have concerning this matter to this office.

One (1) signed original and thirty-nine (39) copies of this transmittal have been provided for your use.

Very truly yours,



Robert F. Janecek  
Nuclear Licensing Administrator  
Boiling Water Reactors

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