

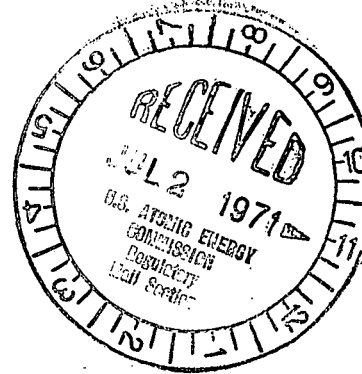
Commonwealth Edison Company

ONE FIRST NATIONAL PLAZA ★ CHICAGO, ILLINOIS

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Dresden Nuclear Power Station
R.R.#1
Morris, Illinois 60450
June 28, 1971



Dr. Peter A. Morris, Director
Division of Reactor Licensing
U.S. Atomic Energy Commission
Washington, D.C. 20545

SUBJECT: LICENSE DPR-19, DRESDEN NUCLEAR POWER STATION UNIT #2,
SECTION 6.6C.1 OF THE TECHNICAL SPECIFICATIONS

Dear Dr. Morris:

This is to report a condition relating to the operation of the station, in which during primary containment local leak rate tests, conducted as required by Section 4.7.A.2.e of the Technical Specifications, a feedwater check valve and the primary containment personnel air lock were found to have leakage rates outside Technical Specifications Limits.

Problem, Investigation and Corrective Action

Local leak rate tests were conducted, as required by Section 4.7.A.2.e of the Technical Specifications, during the period February 22 to May 25, 1971. As a result of these tests, two areas of leakage outside Technical Specification limits were found.

Feedwater Check Valve (220-58B)

Feedwater check valve 220-58B was leak tested at 48 psig and found to be leaking at a rate of approximately 160 SCFH, which is 565% of the allowable limit of 29.381 SCFH for a single isolation valve. Disassembly of the valve for inspection found the disc seat to be in good condition. However, foreign material in the form of rust and crud was found in the area of the seat ring seal. The seating area was cleaned up, machined and lapped to provide for more seating surface, and retested satisfactorily. Leakage following repairs was measured at approximately 7 SCFH.

Primary Containment Personnel Air Lock

Initial testing of the personnel air lock indicated a leakage rate of approximately 82 SCFH, which is 140% of the allowable limit of 58.763 SCFH for a single penetration. The major source of leakage was the exterior upper handwheel stem seal. It is believed that this leakage was probably due to the unusual amount of use during the refueling outage, prior to conduct of the test. The seal was repaired and the lock retested. Leakage was approximately 113 SCFH, but no leakage was observed at the repaired handwheel stem.

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Upon investigation it was found that two drywell air sample lines, which penetrate the containment via the air lock, and which had been installed in the period between lock tests, had piping welds that were incomplete. Since the pipes are open inside the drywell, a leakage path existed from the lock via the leaking welds, to the interior of the primary containment. The sample lines are not yet operational, so the pipe ends inside of the containment were capped as a temporary measure, and the lock retested satisfactorily, exhibiting no measureable leakage.

Sincerely,

H.K. Hoyt wpc
H.K. Hoyt

HKH:rh