LICENSEE EVENT REPORT

CONTROL BLOCK: [ ] (1–6) (PLEASE PRINT ALL REQUIRED INFORMATION)

	LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01	ILDRS2	50-000000-00	4	01
(7–8, 9–14, 15–25, 26–30, 31–32)				

	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 CONT		T	L	050-0237	052576	060476
(7–8)	(57–58)	(59)	(60)	(61–68)	(69–74)	(75–80)

EVENT DESCRIPTION

02 AT 0251 HOURS ON 5-25-76, DURING AUTOMATIC BLOWDOWN SURVEILLANCE TESTING, THE 203-3A
03 RELIEF VALVE REMAINED OPEN, CAUSING A CONTINUOUS BLOWDOWN CONDITION. BY 0300 HOURS, TORUS
04 LEVEL HAD RISEN TO MINUS 1.2 INCHES, A TECH SPEC VIOLATION. WHEN IT BECAME EVIDENT
05 THAT THE VALVE COULD NOT BE CLOSED (AT 0303 HOURS), A MANUAL SCRAM OF THE UNIT WAS
06 INITIATED. AT 0310 HOURS, FULL-FLOW TORUS COOLING WAS INITIATED. MAXIMUM TRANSIENT
(SEE ATTACHED SHEET)

	SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07	SF	E	VALVOP	N	T020	Y
(7–8, 9–10, 11, 12–17, 43, 44–47, 48)						

CAUSE DESCRIPTION

08 THE FAILURE WAS CAUSED BY EXCESSIVE LEAKAGE ON THE PILOT STAGE OF THE VALVE AS DETER-
09 MINED BY THE VENDOR'S TEST PROCEDURE 8.1. TO CORRECT THIS DEFICIENCY, BOTH THE
10 PILOT STAGE AND SECONDARY STAGE OF THE VALVE WERE REPLACED. THIS ACTION RETURNED THE
(SEE ATTACHED SHEET)

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11	C	015	NA	B	NA
(7–8, 9, 10–12, 13–44, 45, 46–80)					

	FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12	Z	Z	NA	NA
(7–8, 9, 10, 11–44, 45–80)				

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
13	000	Z	NA
(7–8, 9–11, 12, 13–80)			

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
14	000	NA
(7–8, 9–11, 12–80)		

OFFSITE CONSEQUENCES

15 NA
(7–8, 9–80)

LOSS OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
16	Z	NA
(7–8, 9, 10–80)		

PUBLICITY

17 NA
(7–8, 9–80)

ADDITIONAL FACTORS

18 NA
(7–8, 9–80)

19
(7–8, 9–80)

NAME: ROBERT W. COEN PHONE: EXT. 443

8103100146

EVENT DESCRIPTION (continued)

conditions with respect to the torus were as follows: at 0340 hours, torus water temperature was 98 degrees F; at 0430 hours, torus level was plus 2.25 inches. During the event, peak drywell pressure was 0.32 psi; peak torus pressure was 0.42 psi. Following the unit shutdown, a primary containment entry was made to visually inspect the torus and to troubleshoot valve 203-3A. No anomalies were noted during either the internal or the external torus inspection. By 0638 hours, torus level was again within Tech Spec limits. This is the first failure of a Target Rock safety/relief valve at Dresden. (50-237/1976-34)

CAUSE DESCRIPTION (continued)

valve leakage to within the vendor's specifications. To further ensure valve operability, the solenoid valve was disassembled, inspected, and replaced following the discovery of foreign matter in the solenoid valve body. After repairs had been completed, the unit was returned to operation on May 26, and the valve was successfully tested at both 150 and 950 psi.

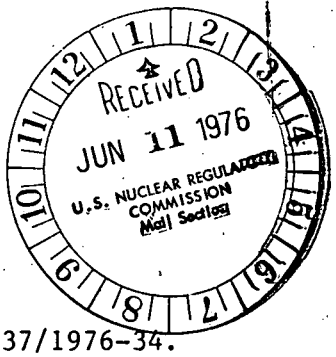
To prevent recurrence, the pilot stage of these valves will be leak-tested during the annual refueling outages. Valve 2-203-3A is a 6-x10- inch combination safety/relief valve, model 7367, manufactured by Target Rock Corp.

Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

BBS Ltr. #439-76

June 7, 1976

Regulatory File Cy.



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

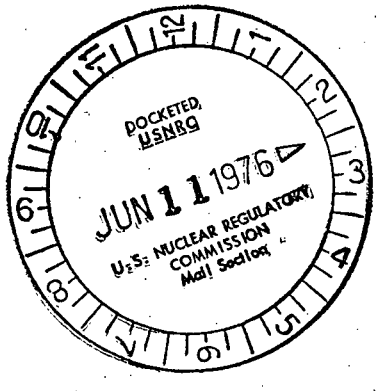
Enclosed please find Reportable Occurrence number 50-237/1976-34. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:smp

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC



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