

LICENSE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 1 L D R S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 57 CAT 58

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 3 7 7 1 1 2 9 7 7 8 1 2 2 9 7 7 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During monthly surveillance DIS 500-7, PS-504C was found to trip at 403 psi. increas-
0 3 ing at SW. Tech Spec 3.1.C. requires that turbine first stage pressure 45% scram
0 4 switches trip in open position <= 400 psi increasing. Safety implications minimal
0 5 since the remaining three switches of the one-out-of-two taken twice logic were oper-
0 6 able and sufficient to allow turbine generator scram logic trips to occur. This event
0 7 has occurred only twice in the past as REF. in DVR #D-12-2-74-32 and /74-62 Docket
0 8 #50/237.
7 8 9

0 9 SYSTEM CODE I A 11 CAUSE CODE E 12 CAUSE SUBCODE E 13 COMPONENT CODE I N S T R U 14 COMP. SUBCODE E 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 18 19 20
17 LER/RO REPORT NUMBER 7 7 21 22 SEQUENTIAL REPORT NO. 0 6 8 24 26 OCCURRENCE CODE 0 3 28 29 REPORT TYPE L 30 31 REVISION NO. 0 32
ACTION TAKEN E 18 19 FUTURE ACTION Z 20 EFFECT ON PLANT Z 21 SHUTDOWN METHOD Z 22 HOURS 0 0 0 0 37 40 ATTACHMENT SUBMITTED N 23 NPRD-4 FORM SUB. Y 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER B 0 6 9 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Cause attributed to instrument drift. Switch reset to 386.5 psi (Dresden setpoint
1 1 limit 387 +/- 5 psi increasing) and checked five times. PS 504C will continue to be
1 2 tested monthly by DIS 500-7. SW. Mfd. by Barksdale Model #B2T-A12SS-GE.
1 3
1 4
7 8 9

1 5 FACILITY STATUS D 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Surveillance Testing 32
7 8 9 10 12 13 44 45 46 80
1 6 ACTIVITY CONTENT Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36
7 8 9 10 11 44 45 80
1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39
7 8 9 11 12 13 80
1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41
7 8 9 11 12 80
1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 80
2 0 ISSUED N 44 DESCRIPTION NA 45
7 8 9 10 80

Carl Lindberg

NRC USE ONLY

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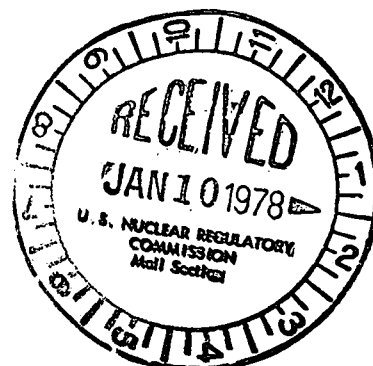
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Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Lankam

REGULATORY DOCKET FILE COPY

December 29, 1977

BBS LTR #1214-77



James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #77-068/03L-0, Docket #050-237
is hereby submitted to your office in accordance with Dresden
Nuclear Power Station Technical Specification 6.6.B.2.(a),
Reactor protection system instrument settings which are found
to be less conservative than those established by the technical
specifications but which do not prevent the fulfillment of the
functional requirements of affected systems.

Arthur M. Roberts
for B.B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:dlz

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

JAN 4 1978

