

# UNDESIRABLE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_ ①

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

① | I | L | D | R | S | 2 | ② | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | ③ | 4 | 1 | 1 | 1 | 1 | ④ | \_\_\_\_\_ | ⑤  
7 8 9 14 15 25 26 30 57 CAT 58

CON'T  
① | L | ⑥ | 0 | 5 | 0 | 0 | 0 | 2 | 3 | 7 | ⑦ | 1 | 1 | 3 | 0 | 7 | 7 | ⑧ | 1 | 2 | 3 | 0 | 7 | 7 | ⑨  
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩  
① | While performing monthly surveillance DIS 700-4 during refueling IRM #13 did not trip  
② | on loss of high voltage or when function switch was not in operate mode. Tech Spec  
③ | Table 3.1.1 requires that IRM trip shall occur when an IRM is inoperative. Safety  
④ | implications minimal since remaining three channels in "A" safety system and all chan-  
⑤ | nels in "B" safety system were operable. This is not a repetitive occurrence.  
⑥ |  
⑦ |  
⑧ |

① | I | A | ⑪ | E | ⑫ | G | ⑬ | I | N | S | T | R | U | ⑭ | Y | ⑮ | Z | ⑯  
9 10 11 12 13 18 19 20

⑰ | 7 | 7 | ⑱ | 0 | 6 | 9 | ⑲ | 0 | 3 | ⑳ | L | ㉑ | 0  
21 22 23 24 26 27 28 29 30 31 32

ACTION TAKEN ⑲ | C | ⑳ | Z | ㉑ | Z | ㉒ | 0 | 0 | 0 | 0 | ㉓ | N | ㉔ | Y | ㉕ | L | ㉖ | G | 0 | 8 | 0 | ㉗  
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ⑳  
① | Cause attributed to defective dual trip module. Failed module was replaced with a  
② | functional spare and DIS 700-4 was successfully completed. IRM's will continue to be  
③ | tested using monthly surveillance DIS 700-4.  
④ |

① | \_\_\_\_\_ | ②  
7 8 9 80

FACILITY STATUS ⑲ | H | ㉑ | 0 | 0 | 0 | ㉒ | NA | ㉓ | B | ㉔ | Surveillance Tests | ㉕  
7 8 9 10 12 13 44 45 46 80

ACTIVITY RELEASED ⑲ | Z | ㉑ | Z | ㉒ | NA | ㉓ | LOCATION OF RELEASE ㉔ | NA | ㉕  
7 8 9 10 11 44 45 80

PERSONNEL EXPOSURES ⑲ | 0 | 0 | 0 | ㉑ | Z | ㉒ | NA | ㉓  
7 8 9 11 12 13 80

PERSONNEL INJURIES ⑲ | 0 | 0 | 0 | ㉑ | NA | ㉓  
7 8 9 11 12 13 80

LOSS OF OR DAMAGE TO FACILITY ⑲ | Z | ㉑ | NA | ㉓  
7 8 9 10 11 12 80

PUBLICITY ISSUED ⑲ | N | ㉑ | NA | ㉓  
7 8 9 10 11 80

8103090630 Carl Lindberg

NRC USE ONLY



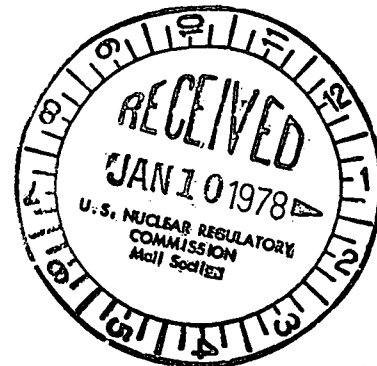
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Dresden Nuclear Power Station  
R.R. #1  
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Telephone 815/942-2920

**REGULATORY DOCKET FILE COPY**

*D. Lanham*

December 30, 1977

BBS LTR #1219-77



James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Reportable Occurrence Report #77-069/03L-0, Docket #050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(a), reactor protection system settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

*Arthur M. Roberts*  
for B.B. Stephenson  
Station Superintendent  
Dresden Nuclear Power Station

BBS:dlz

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
File/NRC

JAN 4 1978

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