NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE	I L D R S 2 (2)
	LICENSE NUMBER	0 0 - 0 0 0 0 0 - 0 0 (3)
	LICENSE TYPE	4 1 1 1 1 (4)
	CAT	(5)
CON'T 0 1	REPORT SOURCE	L (6)
	DOCKET NUMBER	0 5 0 0 0 2 3 7 (7)
	EVENT DATE	1 2 2 2 7 7 (8)
	REPORT DATE	0 1 0 5 7 8 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While performing refueling surveillance DIS 700-12 (Recirc. Flow System) on Unit 2, an
0 3 instrument calib. procedure was misinterpreted and the flow converters were not calib.
0 4 This error resulted in a non-conservative setpoint of the APRM flow bias scram and rod
0 5 block setpoints. Safety Significance minimal since unit conservately operated below
0 6 the 100% flow control line. Also, safety and transient analyses do not assume a flow
0 7 biased scram function. A similar event occurred once before in L.E.R. #77-002/01T-0,
0 8 Docket #050-249.

0 9	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	I A (11)	D (12)	C (13)	I N S T R U (14)	E (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	7 7	—	0 8 1	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
E (18)	G (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	N (25)	G 0 8 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Cause deficient procedure. IM's misinterpreted DIS 700-12 to mean that flow
1 1 converters would be calibrated later when 100% flow was obtained after S/U. Error
1 2 discovered during operations by Nuclear Eng. and flow converters were immediately
1 3 reset using DIP-15. To prevent recurrence a procedural change to ensure conservative
1 4 calib. and review sign-off will be made.

1 5	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
	E (28)	0 8 8 (29)	NA	A (31)	Observation by Nuc. Eng. Dept.

1 6	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

1 7	NUMBER	TYPE	DESCRIPTION (39)
	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

1 8	NUMBER	DESCRIPTION (41)
	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

1 9	TYPE	DESCRIPTION
	Z (42)	NA

PUBLICITY

2 0	ISSUED	DESCRIPTION (45)	NRC USE ONLY
	N (44)	NA	

8103090600

W. Hildy

314

ATTACHMENT TO LICENSEE EVENT REPORT 77-081/01T-0
COMMONWEALTH EDISON COMPANY (CWE)
DRESDEN UNIT 2 (ILDRS-2)
DOCKET #050-237

During routine scheduled calibration of the Unit 2 Recirc. flow system, an instrument calibration procedure was misinterpreted and the flow converters were not calibrated. This omission resulted in a non-conservative setpoint of the APRM flow bias scram and rod-block setpoints. The procedure, DIS-700-12, stated that the final flow converter numbers could not be obtained until 100% flow was reached. The Instrument Department interpreted this to mean that the flow converters would be calibrated when 100% flow was obtained after start up - The actual intent of the procedure was to adjust the flow converters conservatively to insure that when 100% flow was reached that the flow biased trip points would be within the Tech Spec Limit of S = $\leq$ [.65w + 55] and R.B. $\leq$ [.65w + 43].

The error was discovered during routine observation by the Nuclear Engineers and the flow converters were immediately adjusted via DIP-15, which is a procedure intended to make flow converter adjustments during operations.

The corrective action required to prevent a recurrence will be a procedure change which will add a tolerance to the procedure to ensure that the flow converters are calibrated conservatively. Also added will be an approval step by the Nuclear Engineers which will allow them to review the "as-left" calibration of the system immediately upon completion.

Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Lankam

January 5, 1978

BBS LTR #14-78

James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #77-091/01T-0, Docket #050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.1.(b), operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

B.B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:dlz

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

JAN 9 1978