

LICENSEE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_ ①

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L D R S 2 ② 0 0 - 0 0 0 0 0 - 0 0 ③ 4 1 1 1 1 ④ \_\_\_\_\_ ⑤

CON'T. REPORT SOURCE L ⑥ 0 5 0 0 0 2 3 7 ⑦ 1 2 2 3 7 7 ⑧ 0 1 1 3 7 8 ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 While performing monthly surveillance DIS 1600-4, PS 1632A-1 was found to trip at
0 3 55.6" H2O increasing which is not within Tech Spec Limit of <=/= 2 PSI (55.4" H2O,
0 4 Table 3.1.1). Safety implications minimal because remaining three switches of the
0 5 one-out-of-two taken twice logic all tripped within acceptable limits and were suf-
0 6 ficient to provide ECCS initiation upon recognized high drywell condition. A similar
0 7 event occurred only once before as referenced in L.E.R. #76-052/03L-0.

0 8 \_\_\_\_\_

0 9 S F ⑪ E ⑫ E ⑬ I N S T R U ⑭ E ⑮ Z ⑯

17 LER/RO REPORT NUMBER 7 7 EVENT YEAR 7 7 SEQUENTIAL REPORT NO. 0 8 2 OCCURRENCE CODE 3 REPORT TYPE L REVISION NO. 0

ACTION TAKEN E ⑱ Z ⑲ EFFECT ON PLANT Z ⑳ SHUTDOWN METHOD Z ㉑ HOURS 0 0 0 0 ATTACHMENT SUBMITTED N ㉓ NPRD-4 FORM SUB. N ㉔ PRIME COMP. SUPPLIER N ㉕ COMPONENT MANUFACTURER S 3 8 2 ㉖

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 Cause attributed to instrument drift. Switch was reset to 53.2" H2O increasing.
1 1 ECCS drywell high pressure switches will continue to be tested monthly using DIS
1 2 1600-4.
1 3
1 4

1 5 E ㉘ 0 8 5 ㉙ NA ㉚ B ㉛ Surveillance Testing ㉜

1 6 Z ㉝ Z ㉞ NA ㉟ LOCATION OF RELEASE NA ㊱

1 7 0 0 0 ㉡ Z ㉢ NA ㉣

1 8 0 0 0 ㉤ NA ㉥

1 9 Z ㉦ NA ㉧

2 0 N ㉨ NA ㉩ 8103090589 Carl Lindberg ㊰ 289 ㊱

NRC USE ONLY

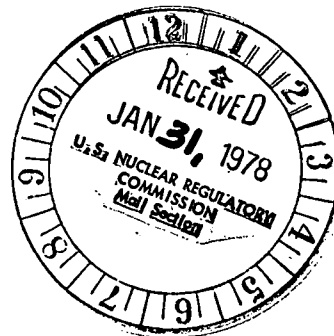
BO 917-926



**Commonwealth Edison**  
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 Telephone 815/942-2920

*D. Lanham*

**REGULATORY DOCKET FILE COPY**



January 13, 1978

BBS LTR #39-78

James G. Keppler, Regional Director  
 Directorate of Regulatory Operations - Region III  
 U.S. Nuclear Regulatory Commission  
 799 Roosevelt Road  
 Glen Ellyn, IL 60137

Reportable Occurrence Report #77-082/03L-0, Docket #050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(a), engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

*B.B. Stephenson*

B.B. Stephenson  
 Station Superintendent  
 Dresden Nuclear Power Station

BBS:d1z

Enclosure

cc: Director of Inspection & Enforcement  
 Director of Management Information & Program Control  
 File/NRC

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JAN 16 1978

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