7-77)	RM 366 U. S. NUCLEAR REGULATORY COMMISSION CENSEE EVENT REPORT
	CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
0 1	1
CON'T 0 1 , 8	REPORT L 6 0 5 0 0 0 2 3 7 7 0 2 2 5 7 8 8 0 3 0 9 7 8 9 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10 During Primary Isolation Valve surveillance DOS 1600-1, continuous air monitor out-
0 3	board isolation valve 9207B failed to close. Inboard isolation valve 9207A, which was
0 4	closed upon discovery of 9207B failure, as required by Technical Specifications, was
0 5	operable: and therefore, this event has little safety significance. One other similar
0 6	event was reported by LER 50-237/77-60.
0 7	L
0 8	
7 8	SYSTEM CAUSE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE P B 10 E 12 E 13 V A L V E X 14 E 15 D 16
	LER/RO EVENT YEAR REPORT NO. 17 REPORT NUMBER 21 22 23 24 26 27 28 29 30 31 32 ACTION FUTURE EFFECT SHUTDOWN HOURS 22 ATTACHMENT FORM SUB. PRIME COMPONENT MANUFACTURER ACTION FUTURE ACTION ON PLANT METHOD HOURS 22 SUBMITTED FORM SUB. SUPPLIER MANUFACTURER ACTION FUTURE COMPONENT SUBMITTED FORM SUB. SUPPLIER MANUFACTURER ACTION FUTURE COMPONENT SUBMITTED FORM SUB. SUPPLIER MANUFACTURER ACTION FUTURE COMPONENT SUBMITTED FORM SUB. SUPPLIER MANUFACTURER ACTION FORM SUB. SUPPLIER MANUFACTURER COMPONENT MANUFACTURER ACTION ON PLANT METHOD HOURS 22 ATTACHMENT FORM SUB. SUPPLIER MANUFACTURER ACTION FUTURE COMPONENT FORM SUB. SUPPLIER FORM SUB. SUPPL
10	Excessive leakage through solenoid held the main valve diaphram open. The pilot valve
11	assembly (ASCO) was replaced and 9207B tested satisfactorily per DOS 1600-1. 9207B
1 2	is a Medel 667ES SIZE 30 gate valve. DOS 1600-1 is performed quarterly and because
1 3	of overall infrequent nature of event, no other corrective action is required.
1 4	80
1 5 7 8	G 28 0 0 0 0 0 0 0 0 0
1 6	CTIVITY CONTENT LEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36 NA NA
7 8	9 10 11 44 45 . 80 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)
7 8	9 PERSONNEL INJURIES 13 80
1 8	NUMBER DESCRIPTION (41) NA NA
7 8	9 11 12 LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION
7 8	9 10 PUBLICITY PUBLICITY NRC USE ONLY
2 0	N 44 NA
	9 10 8/03090492 Parcell 265

Commonweal Edison Dresden Nuclear Power Station R.R. #1 Morris, Illinois 60450 Telephone 815/942-2920

D. Landan)
FILE COPY

March 9, 1978

BBS LTR #254

James G. Keppler, Regional Director Directorate of Regulatory Operations - Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

Reportable Occurrence Report #78-015/03L-0, Docket 050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.b, conditions leading to operation in a degraded mode permitted by a limiting condition for operation.

B. Stephenson

Station Superintendent Dresden Nuclear Power Station

BBS:cac

Enclosure

Director of Inspection & Enforcement Director of Management Information & Program Control File/NRC