

LICENSEE EVENT REPORT

CONTROL BLOCK: [1][2][3][4][5][6] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | I | L | D | R | S | 2 | 2 | 0 | 0 | = | 0 | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
8 9 14 15 25 26 30 57 CAT 58
LICENSEE CODE LICENSE NUMBER LICENSE TYPE

CON'T
01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 3 | 7 | 7 | 0 | 3 | 2 | 7 | 7 | 8 | 8 | 0 | 4 | 0 | 7 | 7 | 8 | 9
8 60 61 68 69 74 75 80
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | While performing surv. DIS 1700-1 (Main Stm Line Hi Rad Scram Isol.) setpoint set
03 | high by 6%. Procedure revision on 8/11/77 generated typographical error in the set-
04 | point formula, however, Tech Spec limit was still correctly shown (Table 3.2.1).
05 | Safety implications to this event were minimized because the incorrectly adj. setpoints
06 | were still within max possible accuracy tolerance of the monitor (GE Model #194X629G7)
07 | & because this instru. is for gross fuel failure. No similar events of this type have
08 | occurred.
8 9

09 | SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
C | D | 11 | A | 12 | X | 13 | I | N | S | T | R | U | 14 | E | 15 | Z | 16
9 10 11 12 13 18 19 20
LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
17 | 7 | 8 | 0 | 2 | 5 | 0 | 1 | T | 0
21 22 23 24 26 27 28 29 30 31 32
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
E | 18 | Z | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | Y | 23 | N | 24 | N | 25 | G | 0 | 8 | 0 | 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | Cause attributed to human error. Trip setpoint had been adjusted above the Tech Spec
11 | limit of </3XNormal as stated in DIS 1700-1. Upon discovery MS high rad monitors
12 | immediately reset to within the Dresden Tech Spec limits and setpoint formula typo-
13 | graphical error was corrected. Because of the singular nature of event no further
14 | action required.
8 9

15 | FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
E | 28 | 0 | 9 | 5 | 29 | NA | 30 | B | 31 | Surveillance Testing | 32
8 9 10 12 13 44 45 46 80
ACTIVITY CONTENT RELEASER OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE
16 | Z | 33 | Z | 34 | NA | 35 | NA | 36
8 9 10 11 44 45 80
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
17 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39
8 9 11 12 13 80
PERSONNEL INJURIES NUMBER DESCRIPTION
18 | 0 | 0 | 0 | 40 | NA | 41
8 9 11 12 80
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
19 | Z | 42 | NA | 43
8 9 10 80
ISSUED DESCRIPTION PUBLICITY
20 | N | 44 | NA | 45
8 9 10 80



Commonwealth Edison
 Dresden Nuclear Power Station
 R.R. #1
 Morris, Illinois 60450
 Telephone 815/942-2920

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April 7, 1978

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James G. Keppler, Regional Director
 Directorate of Regulatory Operations - Region III
 U.S. Nuclear Regulatory Commission
 799 Roosevelt Road
 Glen Ellyn, IL 60137

Reportable Occurrence Report #78-025/01T-0, Docket #050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.1.(b), operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

B. B. Stephenson
 Station Superintendent
 Dresden Nuclear Power Station

BBS:cac

Enclosure

cc: Director of Inspection & Enforcement
 Director of Management Information & Program Control
 File/NRC

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ATTACHMENT TO LICENSEE EVENT REPORT 78-025/01T-0
COMMONWEALTH EDISON COMPANY (CWE)
DRESDEN UNIT 2 (ILDRS2)
DOCKET #050-237

During the quarterly scheduled calibration DIS 1700-1 (Main Steam Line High Radiation Scram and Isolation), the setpoints were set high by 6%. A procedure revision on 8/11/77 generated a typographical error in the setpoint formula. However, the Technical Specification limit was still correctly shown (Table 3.2.1). The Control Systems Technician failed to recognize the incorrectly typed formula and proceeded to adjust the new "As Left" trip setpoints accordingly. The "As Left" setpoints were adjusted by 6% above the Tech Spec limit of $\leq 3XQ$ where $Q =$ Full Power Background Count at time of surveillance. The Main Steam Line Rad Monitors are designed to alert the operator of possible fuel damage, upon an alarm condition. The trip initiates a Group 1 isolation, and reactor scram.

The four monitors are arranged schematically in a one-out-of-two-taken twice logic. The MSL High Rad Monitors are divided into six evenly-spaced decades with a full meter scale of 1×10^6 . The fourth decade (10^3 to 10^4) is the decade of interest and has a documented accuracy of $\pm 20\%$ of true input current. Thus the Safety Implications to this event were minimized because the incorrectly adjusted setpoints were still within the maximum possible accuracy tolerances of the monitor (Model 194X629G7), and the instrument is designed to detect only gross fuel failures. The cause is attributable to human error. The trip setpoint had been adjusted above the Tech Spec limit of $< 3 X Q$ as stated in DIS 1700-1. The immediate corrective action was to perform DIS 1700-1 and to reset the trip setpoints to within the Dresden Tech Spec limits. The setpoint formula typographical error was corrected.