NRC FORM 366 **U.S. NUCLEAR REGULATORY COMMISSION** (7.77)LICENSEE EVENT REPORT CONTROL BLOCK: ①Ľ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 57 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 $\frac{2}{14}$ (2) (0) (0)L 5 0 LICENSEE CODE CON'T 0 5 0 0 0 2 3 7 0 0 4 2 1 7 8 8 0 5 0 5 7 8 DOCKET NUMBER _68 69 EVENT DATE 74 75 REPORT DATE 80 0 1-EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During normal Unit 2 operation and routine Unit 3 Refueling Operations on April 21, 0 2 1978 the NRC requested immediate actions be taken to mitigate the potential for a 03 spurious closure of a tecirculation loop suction galve with a LOCA occurring between 0 4 the doop discharge and suction valves. This situation has not been previously analyzed and 0 5 is reportable per section 6.6.B.1(i) of the Technical Specifications 0 6 0 7 0 8 80 SYSTEM CAUSE CAUSE COMP VALVE CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE X X X X X (14) X (12) X (13) X Z (15) Z (16 (11)18 OCCURRENCE REVISION SEQUENTIAL REPORT EVENT YEAR REPORT NO. CODE LER/RO TYPE NO. (17) REPORT 2 | 8 0 Τ 1 0 NUMBER FUTURE ATTACHMENT SUBMITTED PRIME COMP. COMPONENT ACTION EFFECT ON PLANT SHUTDOWN NPRD-4 HOURS (22) FORM SUB MANUFACTURER 0 0 0 0 (18) Z (19) Z Z (21) 9 9 9 (26) _](20) Y (23) N (24) (25 Ζ CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) On April 21, 1978 to comply with the NRC request a Mod. was initiated for each unit 1 0 which will defeat that portion of the (PCI Loop Select Logic which auto closes the 1 1 Suction valves. This Mod. is presently pending SNED approval. As an interim solution 1 2 both recirc. loop suction valve circuit breakers will be taken 0.0.S. in the open 1 3 position, 4 9 8 80 METHOD OF FACILITY STATUS (30) OTHER STATUS DISCOVERY DESCRIPTION (32) % POWER 9. 8 (28) NRC Notification 5 0.1 NA D (31) -10 CONTENT 80 ACTIVITY AMOUNT OF ACTIVITY (35) RELEASED_OF RELEASE LOCATION OF RELEASE NA Z (33) Z (34) NA 45 10 44 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 01](38) 0(37)ZNA PERSONNEL INJURIES 80 DESCRIPTION (41) NUMBER NA 0 (40) 8 Ω 11 12 80 LOSS OF OR DAMAGE TO FACILITY 43 Z NA (42) 10 PUBLICITY NRC USE ONLY DESCRIPTION (45) ISSUED N (44) NA. 68 69 80 103090461 J. Wujciga 265 NAME OF PREPARER PHONE

ATTACHMENT TO LICENSEE EVENT REPORT 78-028/01T-0 COMMONWEALTH EDISON COMPANY (CWE) DRESDEN UNIT-2 (ILDRS-2) DOCKET #050-237

During normal Unit 2 operation at 820MWe and routine Unit 3 refueling operations on April 21, 1978 the Nuclear Regulatory Commission ((NRC)) requested that adequate action be taken immediately to mitigate the potential for a spurious closure of a recirculation loop suction valve with a LOCA occurring between the loop discharge and suction valves. This situation, which had not been considered in the current LOCA analysis, could decrease the reactor water inventory while reactor pressure remained high. Technical Specification 6.6.B.1.i. requires reporting of performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

On April 21, 1978, to comply with the above NRC request, a modification was initiated for each unit. The modification will defeat only that portion of the Low Pressure Coolant Injection (LPCI) Loop Select Logic which automatically closes the recirculation loop suction valves. The Station Nuclear Engineering Department (SNED) is currently reviewing the modification. The Unit 2 modification will not be completed until a unit outage of sufficient duration occurs following SNED approval. As an interim solution, both recirculation loop suction valve circuit breakers were taken out of service with the valves in the open position. While in this condition, the LPCI system will continue to function as originally analyzed. Operation of the suction valves will be permitted per operating order #27-78 if an operational event occurs which requires the suction valve to be closed. In this case both the loop discharge valve and discharge bypass valve will also be closed thereby isolating the recirculation loop.





Commonwe Edison Dresden Nuclear Power Station R.R. #1 Morris, Illinois 60450 Telephone 815/942-2920

May 5, 1978

BBS 78-799

James G. Keppler, Regional Director Directorate of Regulatory Operations - Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Il 60137

Reportable Occurence Report #78-028/01T-0 Docket #050-237_is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.1.(i). Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

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B. B. Stephenson Station Superintendent Dresden Nuclear Power Station

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Enclosure

cc: Director of Inspection & Enforcement Director of Management Information & Program Control File/NRC

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