



Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

BBS Ltr. #914-74

Dresden Nuclear Power Station
R. R. #1
Morris, Illinois 60450
December 20, 1974

Regulatory

File Cy.

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations-Region III
U. S. Atomic Energy Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



SUBJECT: REPORT OF ABNORMAL OCCURRENCE PER SECTION 6.6.A OF THE TECHNICAL SPECIFICATIONS
PRIMARY CONTAINMENT BELLOWS PENETRATION X-123 LEAKAGE

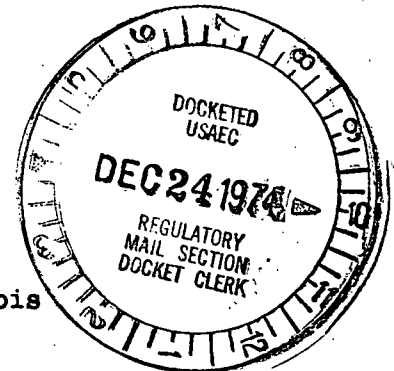
- References: 1) Regulatory Guide 1.16 Rev. 1 Appendix A
- 2) Notification of Region III of AEC Regulatory Operations
Telephone: Mr. P. Johnson at 1545 hours on December 13, 1974
Telegram: Mr. J. Keepler on 0920 hours on December 16, 1974
- 3) Drawing Number: Q 6937

Report Number: 50-237/74-76

Report Date: December 20, 1974

Occurrence Date: December 12, 1974

Facility: Dresden Nuclear Power Station, Morris, Illinois



IDENTIFICATION OF OCCURRENCE

While conducting local leak rate testing the single bellows seal on drywell penetration X-123 (Reactor Building Closed Cooling Water Inlet) what appeared to be excessive leakage was demonstrated. Subsequent investigation showed that the leakage was not in excess of Technical Specifications. However, the event does represent an abnormal degradation of one of the several boundaries designed to contain radioactive materials.

CONDITIONS PRIOR TO OCCURRENCE

At the time of the occurrence, Unit 2 was shutdown and in the REFUEL mode.

12954

8103060701

December 20, 1974

DESCRIPTION OF OCCURRENCE

At 1330 hours on December 12, 1974 the bellows seal on penetration X-123 was one of fourteen bellow seals being simultaneously local leak rate tested. A leakage rate in excess of the test equipment capacity was present. To determine which penetration or penetrations were leaking, the test was repeated for each penetration individually. The leakage through penetration X-123 was found to be in excess of test equipment capacity. The local leak rate test was again repeated using the pressure decay method to determine the leakage rate.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE (Equipment Failure)

After removing the protective shield located around the bellows, one crack and numerous pinholes were found on the first convolution nearest the drywell.

ANALYSIS OF OCCURRENCE

When using the pressure decay method for determining leak rates, the size of the volume being tested must be known. At the time the leak was discovered, the size of the volume was unknown. Once the volume was determined, the actual leakage rate was found to be 21.941 SCFH. The actual leak rate is less than the Technical Specification limit of 29.38 SCFH; therefore, an abnormal occurrence did not take place.

CORRECTIVE ACTION


A representative from Pathway Bellows Incorporated will be coming to the site to examine the defective bellows. The bellows will either be repaired or replaced depending upon the examination results.

The corrective action taken will be reported in the local leak rate test summary at the end of the refueling outage.

FAILURE DATA

To date, the only bellows seal failure was the Unit 2 penetration X-105C which was not a failure of the bellows assembly.

Sincerely,



B. B. Stephenson
Superintendent

BBS:GAR:smp



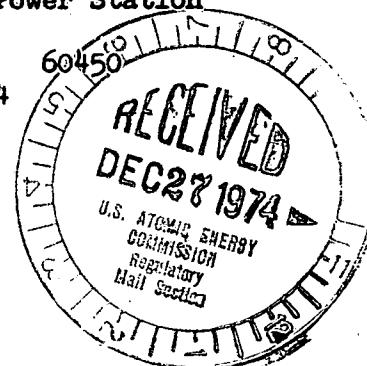
Commonwealth Edison
One First National Plaza, Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

Regulatory

Cy.

BBS Ltr. #914-74

Dresden Nuclear Power Station
R. R. #1
Morris, Illinois 60450
December 20, 1974



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations-Region III
U. S. Atomic Energy Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

SUBJECT: REPORT OF ABNORMAL OCCURRENCE PER SECTION 6.6.A OF THE TECHNICAL SPECIFICATIONS
PRIMARY CONFINEMENT BELLOWS PENETRATION X-123 LEAKAGE

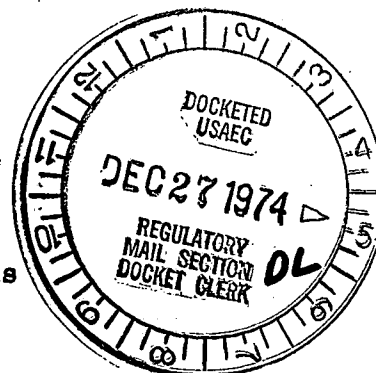
- References: 1) Regulatory Guide 1.16 Rev. 1 Appendix A
- 2) Notification of Region III of AEC Regulatory Operations
Telephone: Mr. P. Johnson at 1545 hours on December 13, 1974
Telegram: Mr. J. Keppeler on 0920 hours on December 16, 1974
- 3) Drawing Number: Q 6937

Report Number: 50-237/74-76

Report Date: December 20, 1974

Occurrence Date: December 12, 1974

Facility: Dresden Nuclear Power Station, Morris, Illinois



IDENTIFICATION OF OCCURRENCE

While conducting local leak rate testing the single bellows seal on drywell penetration X-123 (Reactor Building Closed Cooling Water Inlet) what appeared to be excessive leakage was demonstrated. Subsequent investigation showed that the leakage was not in excess of Technical Specifications. However, the event does represent an abnormal degradation of one of the several boundaries designed to contain radioactive materials.

CONDITIONS PRIOR TO OCCURRENCE

At the time of the occurrence, Unit 2 was shutdown and in the REFUEL mode.

depe
8103060701
DEC 23 1974

13043

December 20, 1974

DESCRIPTION OF OCCURRENCE

At 1330 hours on December 12, 1974 the bellows seal on penetration X-123 was one of fourteen bellow seals being simultaneously local leak rate tested. A leakage rate in excess of the test equipment capacity was present. To determine which penetration or penetrations were leaking, the test was repeated for each penetration individually. The leakage through penetration X-123 was found to be in excess of test equipment capacity. The local leak rate test was again repeated using the pressure decay method to determine the leakage rate.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE (Equipment Failure)

After removing the protective shield located around the bellows, one crack and numerous pinholes were found on the first convolution nearest the drywell.

ANALYSIS OF OCCURRENCE

When using the pressure decay method for determining leak rates, the size of the volume being tested must be known. At the time the leak was discovered, the size of the volume was unknown. Once the volume was determined, the actual leakage rate was found to be 21.941 SCFH. The actual leak rate is less than the Technical Specification limit of 29.38 SCFH; therefore, an abnormal occurrence did not take place.

CORRECTIVE ACTION

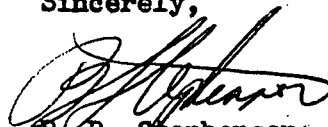
A representative from Pathway Bellows Incorporated will be coming to the site to examine the defective bellows. The bellows will either be repaired or replaced depending upon the examination results.

The corrective action taken will be reported in the local leak rate test summary at the end of the refueling outage.

FAILURE DATA

To date, the only bellows seal failure was the Unit 2 penetration X-105C which was not a failure of the bellows assembly.

Sincerely,


B. B. Stephenson
Superintendent

BBS:GAR:smp