



**Commonwealth Edison**  
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March 27, 1980

Mr. Dennis L. Ziemann  
Operating Reactors Branch #2  
Division of Operating Reactors  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Subject: Status Report on SSRT  
Open Issues  
NRC Docket No. 50-237

References (a): D. L. Ziemann to D. L. Peoples letter of  
January 9, 1980

(b): R. F. Janacek to D. L. Ziemann letter of  
March 4, 1980

Dear Mr. Ziemann:

This letter satisfies the commitment in reference (b) to  
provide a status report for the open SSRT review items for  
Dresden Unit 2.

#### Reactor Vessel Supports

Pursuant to telephone conversations between the NRC,  
General Electric and Commonwealth Edison, General Electric  
proceeded to extract information from their original design  
files. This information was recently transmitted to Commonwealth  
Edison and in turn is being transmitted to Paul O'Connor. The  
information should be transmitted by the end of March 1980.  
Based on the conversation between Dr. John Stevenson and GE we  
believe the information we are transmitting will resolve the  
issue.

#### Reactor Pressure Vessel Internals

Commonwealth Edison had General Electric declassify this  
report on the Dresden 2 reactor internals for use on SEP. The GE  
report number 257HA718 titled Seismic Analysis of Reactor I  
internals for the Dresden II Plant dated 12-24-68 was submitted  
to the NRC in mid February 1980. Based on conversations with Dr.  
Stevenson we believe the report addressed his concerns and as a  
result this item should be resolved.

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### Control Rod Drive Hydraulic Tubing and Supports

We have reviewed the control rod drive racks installed in Dresden Unit 2 and LaSalle County Units 1 and 2 with the intent of qualifying the CRD racks of Dresden 2 through geometric similarity with the LaSalle 1 and 2 CRD equipment. The reference drawings indicate the CRD racks for the two stations are essentially identical; however, the seismic qualification for the LaSalle CRD racks must be reviewed for the seismic requirements of the Dresden Station - Unit 2. This information has not been received yet, but is expected to be provided to Sargent & Lundy by GE for the NSSS seismic qualification being conducted by Sargent & Lundy for the LaSalle County Plants.

### Motor Operated Valves on Small Lines

The work being done for IE Bulletin 79-14 should address the reanalysis of a majority of the eccentric masses of air or motor operated valves on small lines. A list of air or motor operators on small lines has been prepared to check the IE Bulletin 79-14 work to assure all the valves the SSRT is concerned about will be investigated either by the 79-14 work group or via a separate analysis.

### Pipe Support Spacing

The work being done for IE Bulletin 79-14 addresses this issue. The walkdown of the plant is complete and we are currently in the analysis phase of the bulletin's requirements. Commonwealth plans to analyze any significant deficiency found in the Quad Cities and Dresden units first, then we will go back and complete all the required analysis on Dresden 2 first.

### Recirculation Pump Supports

The requested support drawings were transmitted to the NRC in mid February 1980. All the seismic qualification data is included in reference 5.19c "Dresden Unit 2 and 3 Nuclear Plant Earthquake Analysis: Recirculation Loop Piping" by J. A. Blume and Assoc. of the SSRT report, the General Electric "Design Report Recirculation System Dresden II" dated 12/22/69 sent to Dr. Stevenson on February 22, 1979 and the information supplied to D. K. Jabbour on March 19, 1979 and April 4, 1979.

### Battery Racks

Information showing the adequacy of the battery racks was transmitted to the NRC in late December 1979.

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### Cable Trays and Supports

Commonwealth Edison has obtained a copy of the Bechtel Cable Tray and Conduit Raceway Seismic Test Program report. The Bechtel report results are currently being reviewed for applicability to Dresden 2. If the Bechtel report is not applicable we will develop and submit a program for resolving this issue for approval before commencing analytical studies.

### Support of Safety Related Electrical Equipment

A field walkdown inspection of safety related electrical equipment has been completed. The inspection was sufficient to show all equipment except for one 250V DC motor control center has positive anchorage. A repair for the referenced motor control center is being initiated to provide positive anchorage as required by the original design standards. Commonwealth Edison has also initiated a program to review the adequacy of the original anchorage design. This review will be completed by September 1, 1980.

### AC And DC Motor Control Centers

General Electric has recently informed Commonwealth Edison that they could not find any seismic data on the Cutler Hammer 250V DC motor control centers. Commonwealth Edison is currently planning on obtaining an outside testing service to provide required information via test.

General Electric is currently comparing the Dresden 2 MCC's to their current MCC design.

### Instrumentation and Control Room Panels

We have investigated the essential racks and panels for Dresden Station Unit 2 listed in Table 2 of the General Electric Seismic Report dated June, 1970 with Revision 1 dated January 26, 1971. We have found that all panels and racks except instrument rack 902-19 and Hoffman enclosure 2202-22 have had bracing added. The bracing added was not always the same configuration that General Electric recommended in their report. The following descriptions and the attachment will describe and illustrate the way in which the different bracing was added.

Vertical boards 902-15 and 902-17 had a vertical brace added as shown in the attachment except the top was directly bolted to the top of the cabinet by two bolts instead of bolting to a piece of P-1000 unistrut. The vertical brace was fastened to the panel by lateral elements of the type shown in section B-B

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of the attachment. There are five of these lateral elements evenly spaced from top to bottom.

Instrument racks and vertical boards 902-32, 902-33, 902-39, 902-46, and 902-47 had bracing added as illustrated on attachment A. The vertical brace was fastened to the panel by lateral elements of the type shown in section A-A of the attachment. The 1/4" threaded rod used for the lateral elements were not always evenly spaced but were always attached to the panel sections containing HFA relays.

Hoffman enclosures 902-40, 902-41, 2202-15A, and 2202-15B had bracing added to the inside of the enclosures. This added bracing has a vertical brace like the one shown in the attachment and is attached to the top and bottom of the enclosures by the same method shown attaching the vertical brace to the floor in the attachment. There are three evenly spaced vertical braces each having three evenly spaced lateral elements as shown in section A-A attached to the back of the enclosure.

All local racks have bracing added to the panel as recommended in the General Electric report. The difference lies in the fact that the racks are located next to walls, therefore, the bracing is attached to the wall instead of the floor behind the racks.

### Torus Sway Rods

Information showing the torus sway rods have sufficient margin was transmitted to the NRC in late December 1980. The information provided should be sufficient to close this issue.

### Interaction Problems

The action plan Commonwealth Edison has developed and plans to go out for bids on in April 1980 to assure non-category I items will not fall or become dislodged and damage safety related equipment is as follows:

1. A set of general arrangement drawings will be marked to show the location of the safety-related mechanical equipment, electrical equipment, cable trays, and HVAC duct work. A set of piping and instrumentation diagrams will be marked to show the safety related piping.
2. At the field, the safety related equipment (i.e. mechanical, electrical, cable trays, HVAC duct work) will be confirmed and all non-seismically supported

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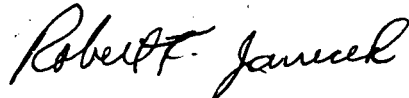
equipment and piping in a 20 foot radius will be noted. This radius may be reduced by engineering judgement once in the field on a case basis due to variation in pipe sizes as an example.

3. From the list of equipment which may fall on safety-related equipment determine which of this non-seismic equipment can cause damage to safety related equipment.
4. Advise CEC Co. in a report of any deficiencies and recommended corrective action.
5. Design seismic supports or relocate the equipment which may cause damage to the safety-related equipment.

The unit is currently undergoing an inspection and cleanup if necessary of areas near safety-related equipment to determine if loose equipment (such as dollies, gas bottle, etc.) could damage them.

One (1) signed original and thirty-nine (39) copies of this transmittal are provided for your use.

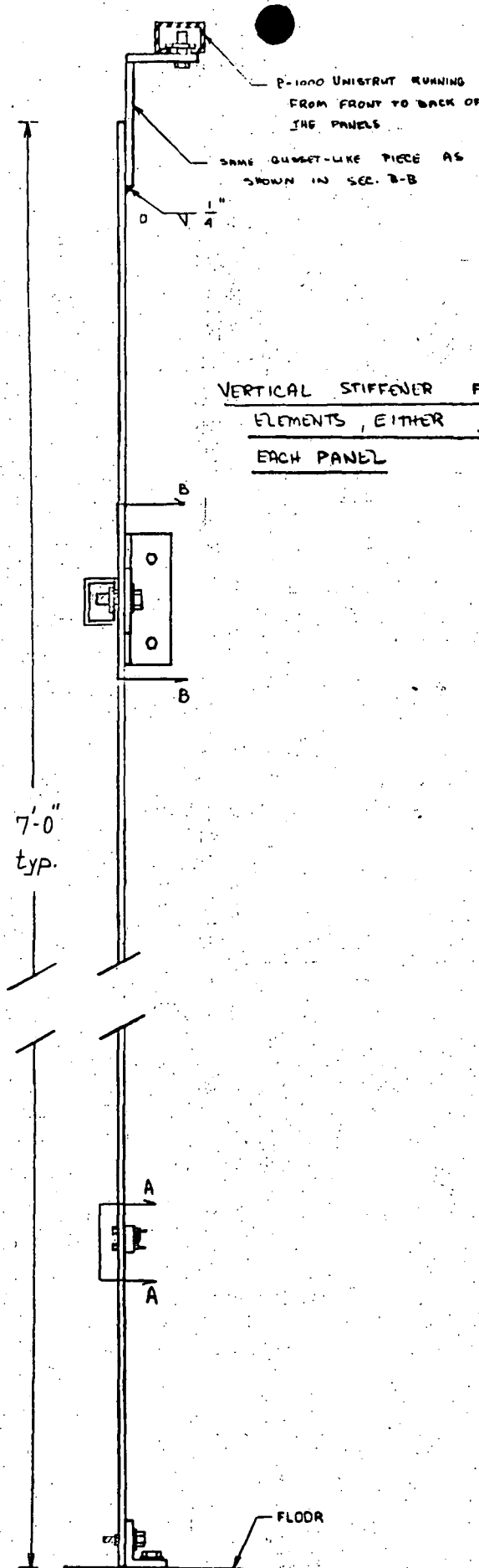
Very truly yours,



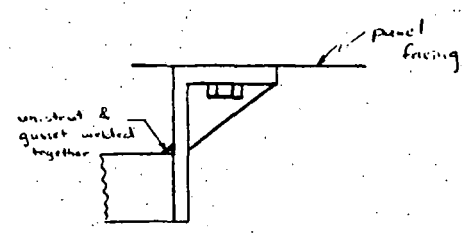
Robert F. Janecek  
Nuclear Licensing Administrator  
Boiling Water Reactors

Attachments

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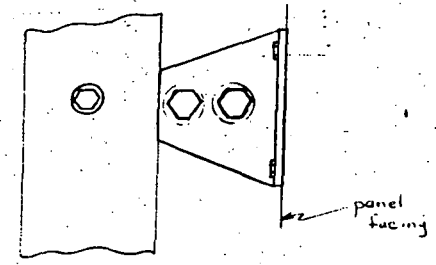
FRONT & BACK END OF UNISTRUT WHICH RUNS ALONG THE PANEL DEPTH WHERE IT IS ATTACHED TO THE PANEL (SIDE VIEW)



VERTICAL STIFFENER FIXED TO THE PANEL BY LATERAL ELEMENTS, EITHER ONE OF TWO TYPES SHOWN IS USED FOR EACH PANEL

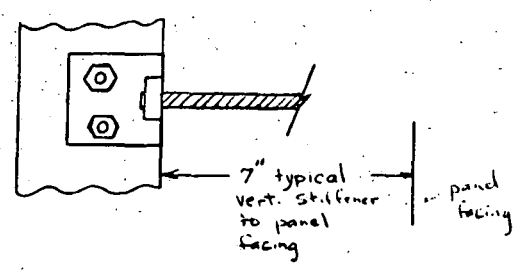
SEC. B-B

THIS LATERAL STIFFENING ELEMENT USES A GUSSET-LIKE PIECE AND P-1000 UNISTRUT TO FIX THE VERTICAL STIFFENING ELEMENT TO THE PANEL



SEC. A-A

THIS STIFFENER USES A ROD TO ATTACH THE VERTICAL STIFFENING ELEMENT TO THE PANEL



NO SCALE

ATTACHMENT