

# UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

### COMMONWEALTH EDISON COMPANY

### DOCKET NO. 50-237

### **DRESDEN NUCLEAR POWER STATION, UNIT 2**

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 168 License No. DPR-19

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A: The application for amendment by the Commonwealth Edison Company (the licensee) dated March 19, 1998, as supplemented March 28, 1998, and April 3, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - Β. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
  - С. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

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# (2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 168, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

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Lawrence W. Rossbach, Project Manager Project Directorate III-2 Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

3.

Date of Issuance: April 10, 1998

# ATTACHMENT TO LICENSE AMENDMENT NO. 168

## FACILITY OPERATING LICENSE NO. DPR-19

# DOCKET NO. 50-237

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE		INSERT
2-1 5-5	· · · · · · · · · · · · · · · · · · ·	2-1 5-5

## 2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

## 2.1 SAFETY LIMITS

#### THERMAL POWER. Low Pressure or Low Flow

2.1.A THERMAL POWER shall not exceed 25% of RATED THERMAL POWER with the reactor vessel steam dome pressure less than 785 psig or core flow less than 10% of rated flow.

APPLICABILITY: OPERATIONAL MODE(s) 1 and 2.

# ACTION:

With THERMAL POWER exceeding 25% of RATED THERMAL POWER and the reactor vessel steam dome pressure less than 785 psig or core flow less than 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours and comply with the requirements of Specification 6.7.

#### THERMAL POWER, High Pressure and High Flow

2.1.B The MINIMUM CRITICAL POWER RATIO (MCPR) shall not be less than 1.08 for Unit 3 and 1.09 for Unit 2 with the reactor vessel steam dome pressure greater than or equal to 785 psig and core flow greater than or equal to 10% of rated flow. During single recirculation loop operation, this MCPR limit shall be increased by 0.01.

<u>APPLICABILITY:</u> OPERATIONAL MODE(s) 1 and 2.

### ACTION:

With MCPR less than the above applicable limit and the reactor vessel steam dome pressure greater than or equal to 785 psig and core flow greater than or equal to 10% of rated flow, be in at least HOT SHUTDOWN within 2 hours and comply with the requirements of Specification 6.7.

#### DRESDEN - UNITS 2 & 3

2-1

# 5.3 REACTOR CORE

#### Fuel Assemblies

5.3.A The reactor core shall contain 724 fuel assemblies<sup>1</sup>. Each assembly consists of a matrix of Zircaloy clad fuel rods with an initial composition of natural or slightly enriched uranium dioxide as fuel material. The assemblies may contain water rods or a water box. Limited substitutions of Zircaloy or ZIRLO or stainless steel filler rods for fuel rods, in accordance with NRC-approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with applicable NRC staff-approved codes and methods, and shown by tests or analyses to comply with all fuel safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in non-limiting core regions.

#### Control Rod Assemblies

5.3.8 The reactor core shall contain 177 cruciform shaped control rod assemblies. The control material shall be boron carbide powder ( $B_4C$ ) and/or hafnium metal. The control rod assembly shall have a nominal axial absorber length of 143 inches.

Operation in all modes with ATRIUM-9B fuel is allowed for Dresden, Unit 3, Cycle 15, and Dresden, Unit 2, Cycle 16, only.

DRESDEN - UNITS 2 & 3