DRESDEN STATION

D3R14 Refueling Outage

In-Service Inspection Summary Report

Commonwealth Edison Co.
P.O. Box 767, Chicago, IL 60690

Dresden Nuclear Power Station
6500 N. Dresden Road, Morris, IL 60450

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Prepared By: Brendan J. Casey , 9-2-97

Reviewed By: [signature] , 9/4/97

Approved By: [signature] , 9/8/97

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Section I
Introduction

The fourteenth Inservice Inspection (ISI) of Dresden Unit 3 was performed during the Spring Refueling Outage, which lasted from March 29, 1997 to June 20, 1997. This was the first outage of the second inspection period of the unit's 3rd 10-year ISI Inspection Interval which commenced on March 1, 1992. The second period commenced on November 1, 1995 and is currently scheduled to end on October 31, 1999.

Raytheon was contracted to perform the non-destructive examinations and General Electric was contracted to perform the reactor vessel visual examinations during the refuel outage. Commonwealth Edision personnel performed the remaining visual examinations. Personnel from Commonwealth Edison's System Materials Analysis Department (SMAD) participated in the inspection to advise on technical problems, review examination results, and evaluate indications.

The Authorized Nuclear Inservice Inspector's (ANII) services were provided by Hartford Steam Boiler Inspection and Insurance Company (HSB). The ANII reviewed procedures, personnel qualifications, instrument and material certifications, and examination results.

All examinations were performed in accordance with the Unit 3 Technical Specifications, the ASME Boiler and Pressure Vessel Code, Section XI, 1989 Edition, and Generic Letter 88-01.

A list of abbreviations used throughout this report can be found in Section IV of this report.

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Section II
Scope of Inspection

Abstract of Examinations

ISI and Augmented Examinations - Table A

Table A contains a list of components examined during D3R14 to satisfy the requirements of ASME Section XI and Generic Letter 88-01. Those items which were examined that required no further evaluation are identified as acceptable. Those items that required further evaluation are discussed in Section III.

Summary of Vessel Interior Examinations - Attachment A

Attachment A contains a summary of examinations performed to satisfy the requirements of ASME Section XI categories BN1, BN2, and various special examination requirements. Details of the examinations, results, and corrective measures are included.

FORM NIS-1 OWNER'S DATA REPORT FOR INSERVICE INSPECTIONS

As required by the Provisions of the ASME Code Rules

1. **Owner:** Commonwealth Edison, One First National Plaza, P.O. Box 767, Chicago, IL 60690-0767

2. **Plant:** Dresden Nuclear Power Station, 6500 N. Dresden Road, Morris, IL 60450

3. **Plant Unit:** Three

4. **Owner Certificate of Authorization:** N/A

5. **Commercial Service Date:** 11/16/71

6. **National Board Number of Unit:** N-139

7. **Components Inspected:** See Section II of attached report (report is 195 total pages).

Component or Appurtenance	Component Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province Number	National Board Number
Class 1 & 2 Systems	Babcock & Wilcox, Barberton, Ohio	610-0111-51	B0082900	N-139

FORM NIS-1 (Back)

8. Examination Dates: 3/29/97 to 6/16/97

9. Inspection Period Identification: Second Inspection Period - From 11/1/95 to 10/31/99

10. Inspection Interval Identification: Third Inspection Interval - From 3/1/92 to 2/28/02

11. Applicable Edition of Section XI 1989 Edition with No Addenda

12. Date/Revision of Inspection Plan: 10/17/94 - Revision 4

13. Abstract of Examinations and Tests. Include a list of examinations and tests and a statement concerning status of work required for the Inspection Plan.

See Attached Sections II and III

14. Abstract of Results of Examinations and Tests.

See Attached Sections II and III

15. Abstract of Corrective Measures.

See Attached Sections III and V

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (if applicable): N/A **Expiration Date:** N/A

Date: 9-10 1997 **Signed For:** Commonwealth Edison Company

By: Brendan J. Casey Site Engineering ISI Coordinator

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois and employed by HSBI & I Co. of Hartford, Connecticut have inspected the components described in this Owner's Report during the period from 3/29/97 to 6/16/97, and state that to the best of my knowledge and belief, the owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in the Owner's Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Brent J. Carney

Inspector's Signature

Commissions: NB7742NISB, IL932 Date: 9-11 19 97

National Board, State, Province, and Endorsements

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Section II
Scope of Inspection

ISI and Augmented Examinations
Table A

Category	Item	Augment	System	Line	Component	Type	Exam	Credit	Results
BA	B1.21	N/A	RPV	RPV UPP HD	3-THD-DA	CIRC	UT	XI	Acceptable.
BA	B1.21	N/A	RPV	RPV UPP HD	3-THD-DB	CIRC	UT	XI	Acceptable.
BA	B1.21	N/A	RPV	RPV UPP HD	3-THD-DC	CIRC	UT	XI	Acceptable.
BA	B1.22	N/A	RPV	RPV UPP HD	3-THD-M1	MERID	UT	XI	Acceptable.
BA	B1.22	N/A	RPV	RPV UPP HD	3-THD-M2	MERID	UT	XI	Acceptable.
BA	B1.22	N/A	RPV	RPV UPP HD	3-THD-M3	MERID	UT	XI	Acceptable.
BA	B1.40	N/A	RPV	RPV UPP HD	3-THD-FLGA	THD-FLG	MT	XI	Acceptable.
							UT	XI	
BA	B1.40	N/A	RPV	RPV UPP HD	3-THD-FLGB	THD-FLG	MT	XI	Acceptable.
							UT	XI	
BA	B1.40	N/A	RPV	RPV UPP HD	3-THD-FLGC	THD-FLG	MT	XI	Acceptable.
							UT	XI	
BA	B1.40	N/A	RPV	RPV UPP HD	3-THD-FLGD	THD-FLG	MT	XI	See Section III
							UT	XI	
BA	B1.40	N/A	RPV	RPV UPP HD	3-THD-FLGE	THD-FLG	MT	XI	See Section III
							UT	XI	
BA	B1.40	N/A	RPV	RPV UPP HD	3-THD-FLGF	THD-FLG	MT	XI	See Section III
							UT	XI	
BD	B3.90	N/A	RPV	RPV UPP HD	N18A-2	NOZ-RPV	UT	XI	Acceptable.
BD	B3.90	N/A	RPV	RPV UPP HD	N18B-2	RPV-NOZ	UT	XI	Acceptable.
BD	B3.90	N/A	RPV	RPV UPP HD	N8-2	RPV-NOZ	UT	XI	Acceptable.
BF	B5.10	GL88-01 D	RHS	0304-6	N18A-3	SE-NOZ	UT	88	Acceptable.
BF	B5.10	GL88-01 D	RHSP	RH SPARE	N18B-3	NOZ-SE	UT	88	Acceptable.
BF	B5.10	GL88-01 D	RHV	0215-4	N8-3	NOZ-SE	UT	88	Acceptable.
BF	B5.130	GL88-01 D	RHV	0215-4	4-1	FLG-P	UT	88	Acceptable.
BF	B5.130	GL88-01 A	SDC	1001A-16	16-48	VLV-EL	UT	OR	Acceptable.
BF	B5.130	GL88-01 A	SDC	1001B-16	16-11	VLV-P	UT	OR	Acceptable.
BF	B5.150	N/A	RVBD	0207-2	2-5	SWE-P	PT	XI	Acceptable.
BF	B5.20	N/A	LVLA	LVLA	N16A-3	NOZ-SE	PT	XI	Acceptable.
							UT	OR	
BF	B5.20	N/A	LVLB	LVLB	N16B-3	NOZ-SE	PT	XI	Acceptable.
							UT	OR	
BF	B5.20	N/A	SBLC	1102-1.5	N12-3	SE-NOZ	PT	OR	Acceptable.
							UT	OR	
BF	B5.20	N/A	UVLA	UVLA	N13A-3	NOZ-SE	PT	XI	Acceptable.
							UT	OR	

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BF	B5.20	N/A	UVLB	UVLB	N13B-3	NOZ-SE	PT UT	XI OR	Acceptable.
BG1	B6.20	N/A	RPV	RPV UPP HD	HD STUDS IN PLC (92)	FLGBLT	UT	OR	Acceptable.
BG1	B6.40	N/A	RPV	RPV UPP HD	FLG THRDS (92)	FLGBLT	UT	XI	Acceptable.
BJ	B9.11	GL88-01 D	RHS	0304-6	6A-1	FLG-SE	UT	88	Acceptable.
BJ	B9.11	GL88-01 D	RHSP	RH SPARE	6B-1	SE-FLG	UT	88	Acceptable.
BJ	B9.11	GL88-01 D	RHV	0215-4	4A-1(A)	SE-FLG	UT	88	Acceptable.
BJ	B9.21	N/A	UVLA	UVLA	UVLA2-1	SE-P	PT	XI	Acceptable.
BJ	B9.40	N/A	LVLB	LVLB	LVLB2-2	P-SWR	PT	XI	Acceptable.
BM2	B12.50	N/A	RWCU	1201-8	MO-3-1201-1	VLV	VT-3/4	XI	Acceptable.
BP	B15.OT	N/A	N/A	TEST BLOCK	RCPB	N/A	VT-2	XI	See Section III
CF1	C5.11	GL88-01 C	ISCOSS	1302-14	14-7	P-TEE	PT UT	XI XI88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302-14	14-8	TEE-RED	UT	88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302-14	14-9	TEE-RED	UT	88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302A-12	12-1	RED-P	PT UT	XI XI88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-1	RED-P	PT UT	XI XI88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-1.1	P-P	UT	88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-2	P-P	UT	88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-3	P-EL	UT	88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-4	EL-P	UT	88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-5	P-EL	PT UT	XI XI88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-6	EL-P	UT	88	Acceptable.
CF1	C5.11	GL88-01 C	ISCOSS	1302B-12	12-7	P-SE	UT	88	Acceptable.
CH	C7.OT	N/A	N/A	TEST BLOCK	03A1	N/A	VT-2	XI	See Section III
CH	C7.OT	N/A	N/A	TEST BLOCK	15A1	N/A	VT-2	XI	Acceptable.
DB	D2.IA	N/A	CCSWBD	1510-16	M-1200D-108	IWA	VT-3/4	XI	Acceptable.
DB	D2.IA	N/A	SRVDA	3019A-8	M-1143 SHT 24	IWA	VT-3/4	OR	Acceptable.

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ISI and Augmented Examinations
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Category	Item	Augment	System	Line	Component	Type	Exam	Credit	Results
FA	F1.30	N/A	CCSWAD	1514D-8	M-1200D-259	CL 3 SUP	VT-3/4	OR	See Section III
FA	F1.30	N/A	CCSWBD	1510-16	M-1200D-108	CL 3 SUP	VT-3/4	XI	Acceptable.

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Summary of Vessel Interior Examinations
Attachment A

During the D3R14 refueling outage, comprehensive visual and ultrasonic examinations of reactor vessel internal components were conducted to ensure the continued integrity of the RPV internals. These examinations also served to meet ASME Code and augmented inspection requirements. The specific components examined, the methods utilized, and the examination results are provided below.

Core Shroud Inspections

The core shroud examinations were performed in conjunction with a comprehensive shroud repair to assure that structural integrity of the core shroud is maintained. The examinations of the reactor core shroud were performed in accordance with ComEd's commitment to NRC Generic Letter 94-03 and BWRVIP guidelines.

The examinations of the core shroud consisted of ultrasonic examination (UT) and enhanced visual examination (EVT-1) of the shroud vertical welds, along with enhanced visual examination of the ring segment welds and remaining design reliant welds and structures. The ultrasonic examinations were performed in accordance with the BWRVIP "Standards for Ultrasonic Examination of Core Shroud Welds" and the visual examinations were performed in accordance with the BWRVIP "Standards for Visual Inspections of Core Shrouds". Additionally, eddy current and ultrasonic examinations were used as aids in identifying the location of ring segment welds associated with the shroud head flange ring, the top guide support ring, and the core plate support ring.

In response to the recent discovery of extensive shroud vertical weld cracking at a domestic BWR, ComEd opted to significantly expand the initial examination scope over that proposed in the original commitment to the NRC. In the original Inspection Plan, ComEd committed to performing enhanced visual examination of 25% of the equivalent length of all vertical welds from either the inner diameter (ID) or outer diameter (OD) surface, along with enhanced visual examination of 25% of the ring segment welds from all accessible surfaces. The expanded examination scope included ultrasonic and enhanced visual examination of the accessible length of each shroud vertical weld (12 total) from the shroud OD, along with enhanced visual examination of each ring segment weld (16 total) from all accessible surfaces.

The following is a summary of the core shroud examination scope and results. Details of the specific areas examined and the results of the examinations are presented in Table 1 and Figure 1.

- The ultrasonic examination scope consisted of shroud vertical welds V5 through V7 (located between the H1 and H2 circumferential welds), V14 through V19 (located in the beltline region between circumferential welds H3 and H5), and V26 through V28 (located between circumferential welds H6 and H7). A total of 551 inches of the 760 inches of total cumulative vertical weld length (or 72.5%) was examined by at least one search unit from one side of the weld. Individual UT length coverage of all 12 vertical shroud welds ranged from between 34% and 96% per weld. These examinations resulted in no reportable indications.

- The enhanced visual examination scope consisted of the ring segment welds associated with the shroud head flange ring (V1 through V4), top guide support ring (V8 through V13), and core plate support ring (V20 through V25); the H8 and H9 circumferential welds at the shroud repair hardware

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locations; and the OD surface of all 12 vertical shroud welds on the side of the weld opposite from the UT search unit. These examinations resulted in no reportable indications in the area of interest. However, circumferential cracking associated with the H2, H3 and H5 welds, as well as vertically oriented cracking in the vicinity of the V23 and V25 core plate ring segment welds was identified during performance of the ring segment weld examinations. The H2, H3 and H5 welds were not part of the core shroud examination scope because the installed comprehensive shroud repair was designed to structurally replace circumferential shroud welds H1 through H7. Therefore, the observed circumferential cracking has no adverse impact on core shroud structural integrity.

The vertically oriented cracking identified in the vicinity of the V23 and V25 ring segment welds was contained in an area of heavy localized grinding on the OD surface of the ring. The cracking is confined to this area of localized grinding and branches down from the circumferentially oriented cracking in the lower heat affected zone (HAZ) of the H5 weld (see Figure 2).

It is important to note that the ring segment welds cannot be identified visually, as all surfaces of the ring were machined after the six individual segments were welded together. As a result of this machining process, heavy circumferential machining grooves are evident on the majority of the ring surface, except where localized grinding was performed. In order to verify the location of the ring segment welds, eddy current examination was performed and a review of archived OD tracker ultrasonic examination data from the D3R13 outage was reviewed. Because the ring segment welds cannot actually be seen, the examination of each weld area included all of the area between two readily discernible landmarks encompassing the azimuth identified for the weld location (i.e., between two jet pump pairs, between two shroud head bolt lug sets, etc.).

Supplemental ultrasonic examination was performed on the vertically oriented cracking in these two areas in order to quantify flaw depths and to determine if the cracking was actually associated with the heat affected zones of the V23 and V25 welds, rather than simply an axial component of the H5 circumferential crack that propagated into the heavily ground area of the ring. The flaws were sized as follows:

V23 - 2.8 inches in vertical length extending downward from the H5 circumferential weld. Maximum radial through wall depth was 0.50 inches at intersection with circumferential crack, with depths ranging between 0.25 inches and 0.40 inches elsewhere.

V25 - Exact vertical length could not be determined due to shallow depth and flaw orientation, however, the area of flaw length sized was 1.2 inches in vertical length extending downward from the H5 circumferential weld. Maximum radial through wall depth was 0.25 inches at intersection with circumferential crack, with depths ranging between 0.15 inches and 0.25 inches elsewhere.

The UT technique employed was only able to scan a five inch area adjacent to the cracking due to interference with the jet pump mixers. However, these scans did not reveal any evidence of a ring segment weld in the ground areas containing the vertically oriented flaws, although the UT technique should have seen evidence of the weld, if it were present.

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Based upon the above, it is believed that the vertically oriented flaws identified in the vicinity of the V23 and V25 ring segment welds are simply axial components of the circumferential cracking contained in the lower heat affected zone of the H5 weld as a result of the surface cold working introduced by heavy localized grinding.

Additionally, even if it were assumed that the vertically oriented cracking was associated with the V23 and V25 ring segment welds, the radial depth of the cracking is shallow and structural margins are ensured for a minimum of 4 operating cycles after applying a radial crack growth rate of $5x10^{-5}$ inches per hour (reference NDIT No. S040-DH-0453, and S&L Calc No. CMED-060298). The core plate support ring is 4.0 inches thick with a radial width of 10.68 inches, where only an area of 7.4 square inches of unflawed ligament is required for the ring to maintain full structural integrity under all design basis and beyond events.

In summary, the examinations of the core shroud design reliant structures performed at Dresden Unit 3 resulted in no indications of cracking that compromise the structural integrity of the core shroud design reliant structures.

Core Spray Inspections

The internal core spray examinations were performed in accordance with the recommendations provided in BWRVIP document BWRVIP-18, BWR Core Spray Internals Inspection and Evaluation Guidelines. The inspection scope consisted of automated ultrasonic examination of all core spray piping welds from the junction box at the RPV nozzle to the downcomer connection at the shroud. Where access restrictions prevented 100% ultrasonic coverage of a given weld, supplemental enhanced visual examination was performed to ensure 100% coverage of the weld, where possible. Enhanced visual examination was also performed on the elbow to shroud pipe welds (P4d) and the shroud pipe to collar welds (P8a), because the ultrasonic techniques employed at these locations have not yet been fully qualified per BWRVIP-03, Reactor Pressure Vessel and Internals Examination Guidelines. Additionally, modified VT-1 visual examinations were performed on all eight core spray wall brackets.

Internal to the shroud, enhanced visual examinations were performed on the core spray sparger tee-box cover plate welds, sparger to tee-box branch connection welds, and sparger end cap welds. Also, modified VT-1 examinations were performed on the sparger nozzles, piping, brackets, and gusset welds.

The examinations conducted identified and sized the two existing flaws in the downcomer pipe to lower elbow welds (P4c), on the 110° and 290° azimuth downcomers. Additionally, three previously undetected flaws were identified and sized in the shroud penetration thermal sleeve collar near the P8a welds. All three of the shroud pipe to collar flaws are located in the collar side of the weld, approximately 0.50" back from the face of the collar. The flaws are located on the 80°, 110°, and 260° downcomers. Specific details of the component geometry and flaw locations are as depicted in Figures 3 through 7.

An evaluation of the core spray flaws was conducted by Sargent & Lundy utilizing limit load analysis techniques (Report Number SL-5130, Rev 1, Dresden Unit 3 Core Spray Flaw Evaluation Report). The results of this analysis demonstrate that the core spray downcomer piping is capable of withstanding all normal operating and design basis loading conditions in its current degraded condition for a minimum of

67 months of hot operation. Consequently, no repairs were implemented during this outage. The condition of the degraded core spray welds will continue to be monitored by following the recommendations provided in BWRVIP-18, BWR Core Spray Internals Inspection and Evaluation Guidelines, during subsequent outages.

Jet Pump Hold Down Beam Ultrasonic Examinations

An ultrasonic examination of nineteen of the twenty jet pump hold down beams was conducted in accordance with NDT-C-29, utilizing the new examination fixture purchased from Siemans Power Corporation. This fixture is capable of detecting flaws in the bolt area of the beam as well as in the engagement ears. One beam (jet pump #8) could not be examined utilizing the new fixture due to interference with the lock plate, which was deformed during a previous outage. Also, the examination identified a crack indication in the shroud side engagement ear of jet pump beam #13. Both the #8 and #13 beams were replaced with BWR 4 style beams by personnel from GE Nuclear Energy under Work Request 970036394.

Jet Pump Restrainer Set Screw Inspections

While performing as-found examinations in preparation for the jet pump #13 beam replacement, crack indications were identified in the vessel side restrainer set screw mounting block fillet welds and a gap was found between the set screw and mixer. Review of past history revealed that the jet pump #13 mixer had ejected during operation in 1980 as a result of a failed jet pump beam. Cracking of a similar nature on the shroud side set screw mounting block was identified at that time and the block and swing gate was replaced.

Examinations of the mixer, wedge assembly, and set screw area revealed no signs of damage or contact between the set screw and mixer. Additionally, the riser braces associated with this jet pump were examined and showed no signs of degradation. Also, the set screw mounting block fillet welds for jet pumps #11 through #20 ("B" Loop) were examined with no additional problems noted.

Based on the above, this condition is believed to be a pre-existing condition associated with the jet pump mixer ejection of 1980.

An evaluation of the acceptability of continued operation with this condition was performed by GE Nuclear Energy (reference GENE Report No. GENE-B13-01869-051-1, dated May 1997). This evaluation consisted of performing analysis of the jet pump with one unsupported mixer and subjecting it to normal, upset and faulted loads. The resulting stresses were combined and compared to the allowable values for each load combination. Based upon the results of this analysis it was concluded that structural integrity of the jet pump is assured and that continued operation of jet pump #13 with an unsupported mixer does not impact safe operation of the plant.

Jet Pump Riser Weld Inspections

A visual examination of the jet pump riser welds was conducted utilizing underwater video cameras in accordance with DTS 0200-02 and the recommendations contained in GE SIL No. 605. The examination

consisted of enhanced visual examination of all accessible areas of the thermal sleeve to elbow welds, elbow to riser welds, and riser to transition piece welds on all ten jet pump risers.

No adverse conditions were noted during the course of these examinations.

Shroud Head Bolt Ultrasonic Examinations

An ultrasonic examination of all 48 shroud head bolts was conducted in accordance with NDT-C-49. This examination identified a crack indication in 1 previously unflawed bolt and reconfirmed the crack indications in eleven previously flawed bolts. Additionally, crack indications were not reconfirmed in four bolts that had been called out as containing flaws during previous examinations, and the bolts were declared sound. None of the crack indications observed were through wall, as evidenced by the presence of a back signal from the end of the bolt. Per previous analysis (Dresden Engineering DOC ID# 4866006, dated 2-23-96), only 12 bolts, evenly spaced, are required to withstand all operating and design basis loading conditions, provided that certain spacing requirements are met. Based upon this analysis, and the as-found distribution of flawed shroud head bolts, no replacements or redistribution of existing bolts was required due to the flaws identified.

However, during reassembly of the shroud head, the keepers associated with two additional bolts were identified as being in the retracted position, and efforts to re-engage them were unsuccessful. Per the engineering analysis discussed previously (DOC ID# 4866006), bolts with retracted keepers are considered to be non functional, since it is possible that they could disengage from their associated lug set during operation. When the location of the two non functional bolts was combined with the location of the flawed bolts, the separation criteria required by the existing analysis was not met.

An evaluation of the acceptability of continued operation with the existing configuration of flawed and non functional shroud head bolts was performed by GE Nuclear Energy (reference GENE Report No. GENE-B13-01869-068, dated June 1997). This evaluation consisted of performing analysis of the existing bolt pattern and subjecting it to normal, upset and faulted loads. The resulting stresses on the shroud head bolts and their associated lugs were then evaluated. Based upon the results of this analysis it was concluded that the existing configuration of shroud head bolts is adequate to withstand all operating and design basis loading conditions, and that structural integrity of the remaining unflawed shroud head bolts is not compromised. As such, continued operation with the existing configuration of flawed and non functional shroud head bolts does not impact safe operation of the plant.

See Figure 8 for the "as-found" / "as-left" distribution of flawed and non functional shroud head bolts.

Lower Plenum Inspections

During the bottom head drain line unplugging project, access to the lower plenum was gained through the removal of two control rod guide tubes at the center of the vessel. A visual examination of the components made accessible through removal of the control rod guide tubes was conducted utilizing underwater video cameras in accordance with DTS 0200-02. The specific components examined included accessible portions of the following components:

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- 2 incore housing to vessel bottom head welds
- 2 incore guide tube to housing welds
- 2 incore guide tube stabilizers
- Core plate stiffener beam to core plate stitch welds in the area adjacent to one removed control rod guide tube
- Core plate stiffener rod to beam (or pipe sleeve) welds in the area adjacent to one removed control rod guide
- 2 CRD housing to CRD stub tube welds
- 2 CRD stub tube to vessel bottom head welds
- Vessel bottom head cladding adjacent to the bottom head drain

This was a "best effort" examination in very difficult to reach areas. No adverse conditions were noted during the course of these examinations.

Top Guide Inspections

A visual examination of the top guide was conducted utilizing underwater video cameras in accordance with DTS 0200-02 and the recommendations contained in BWRVIP Document BWRVIP-26, BWR Top Guide Inspection and Flaw Evaluation Guidelines. The specific components examined included all four alignment pin assemblies, along with approximately 24 inches of the top guide rim to lower ledge weld adjacent to the four alignment pin assemblies.

This was a "best effort" examination in very difficult to reach areas. However, where possible, examinations of weld heat affected zones were performed using enhanced visual techniques capable of discerning a 0.5 Mil fine wire placed against the examination surface. No adverse conditions were noted during the course of these examinations.

Steam Dryer Inspections

A visual examination of the steam dryer was conducted utilizing underwater video cameras in accordance with DTS 0200-02. The specific components examined included all 4 lifting eye assemblies, all 12 drain channels, tie bar assemblies, upper support ring, upper support ring to skirt weld, vertical skirt welds, guides, lower support ring to skirt weld, dryer bank assembly welds, and all 4 steam dryer hold down assemblies.

This examination identified cracking and/or failure of the lifting lug centering rings and gussets at three of the four lifting lug locations. These components have no structural function, rather, they were used to align the lifting lugs for initial installation (reference Dresden Design Engineering Document ID No. 5389574, dated 4-10-97).

All four centering rings and gussets were removed under Work Request 970041070, to eliminate the potential for loose parts.

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Steam Separator Inspections

A visual examination of the steam separator was conducted utilizing underwater video cameras in accordance with DTS 0200-02. The specific components examined included all 4 lifting eye assemblies and attachment welds, peripheral standpipes and assembly welds, tie bars and attachment welds, and the shroud head flange area.

This examination identified a misalignment (bending) between the lifting lug eye and the rod of the lifting lug assembly at the 184.5° location. A detailed examination of the lifting eye attachment weld to the lifting lug rod revealed a small linear indication located at the toe of the attachment weld. The indication was not crack like in nature and was evaluated as a base metal anomaly. These conditions were evaluated by Dresden Design Engineering and found to be acceptable as is (reference Dresden Design Engineering Document ID No. 5405084, dated 5-6-97).

Table 1
Dresden Unit 3 Core Shroud Examination Summary

Component	Area Inspected	Inspection Results
Shroud Head Flange Ring Segment Welds: V1 Through V4	Enhanced visual examination of ~ 6" to 12" length of ring material encompassing each weld. Inspected OD, ID and top of ring.	No Reportable Indications
Top Guide Support Ring Segment Welds: V8 Through V13	Enhanced visual examination of ~ 6" to 12" length of ring material encompassing each weld. Inspected OD and ID of ring.	No Reportable Indications in Area of Interest
Core Plate Support Ring Segment Welds: V20 Through V25	Enhanced visual examination of ~ 6" to 12" length of ring material encompassing each weld. Inspected OD of ring. Supplemental UT examination of V23 and V25	See Note 1
Vertical Welds V5 Through V7 (Between H1 & H2)	Ultrasonic examination of between 34% and 69% of the length of each weld. Enhanced visual examination of entire weld length from shroud OD on side of weld opposite of UT search unit.	No Reportable Indications
Vertical Welds: V14 Through V19 (Beltline Area)	Ultrasonic examination of between 40% and 95% of the length of each weld. Enhanced visual examination of entire weld length from shroud OD on side of weld opposite of UT search unit.	No Reportable Indications
Vertical Welds: V26 Through V28 (Between H6 and H7)	Ultrasonic examination of between 47% and 75% of the length of each weld. Enhanced visual examination of entire weld length from shroud OD on side of weld opposite of UT search unit.	No Reportable Indications
Jet Pump Support Plate to Shroud Support Ring Weld H8	Enhanced visual examination of ~ 12" of weld in area of repair hardware attachments at 4 locations: 20°, 110°, 200°, and 290° azimuths.	No Reportable Indications
Jet Pump Support Plate to RPV Weld H9	Enhanced visual examination of ~ 12" of weld in area of repair hardware attachments at 4 locations: 20°, 110°, 200°, and 290° azimuths.	No Reportable Indications

Note 1: Vertically oriented cracking was identified visually in the vicinity of the V23 and V25 ring segment welds. Supplemental UT sizing provided the following information:

V23 - 2.8 inches in vertical length extending downward from the H5 circumferential weld. Maximum radial through wall depth was 0.50 inches at intersection with circumferential crack, with depths ranging between 0.25 inches and 0.40 inches elsewhere.

V25 - Exact vertical length could not be determined due to shallow depth and flaw orientation, however, the area of flaw length sized was 1.2 inches in vertical length extending downward from the H5 circumferential weld. Maximum radial through wall depth was 0.25 inches at intersection with circumferential crack, with depths ranging between 0.15 inches and 0.25 inches elsewhere.

Figure 1

Dresden Unit 3 Core Shroud Examination Roll Out

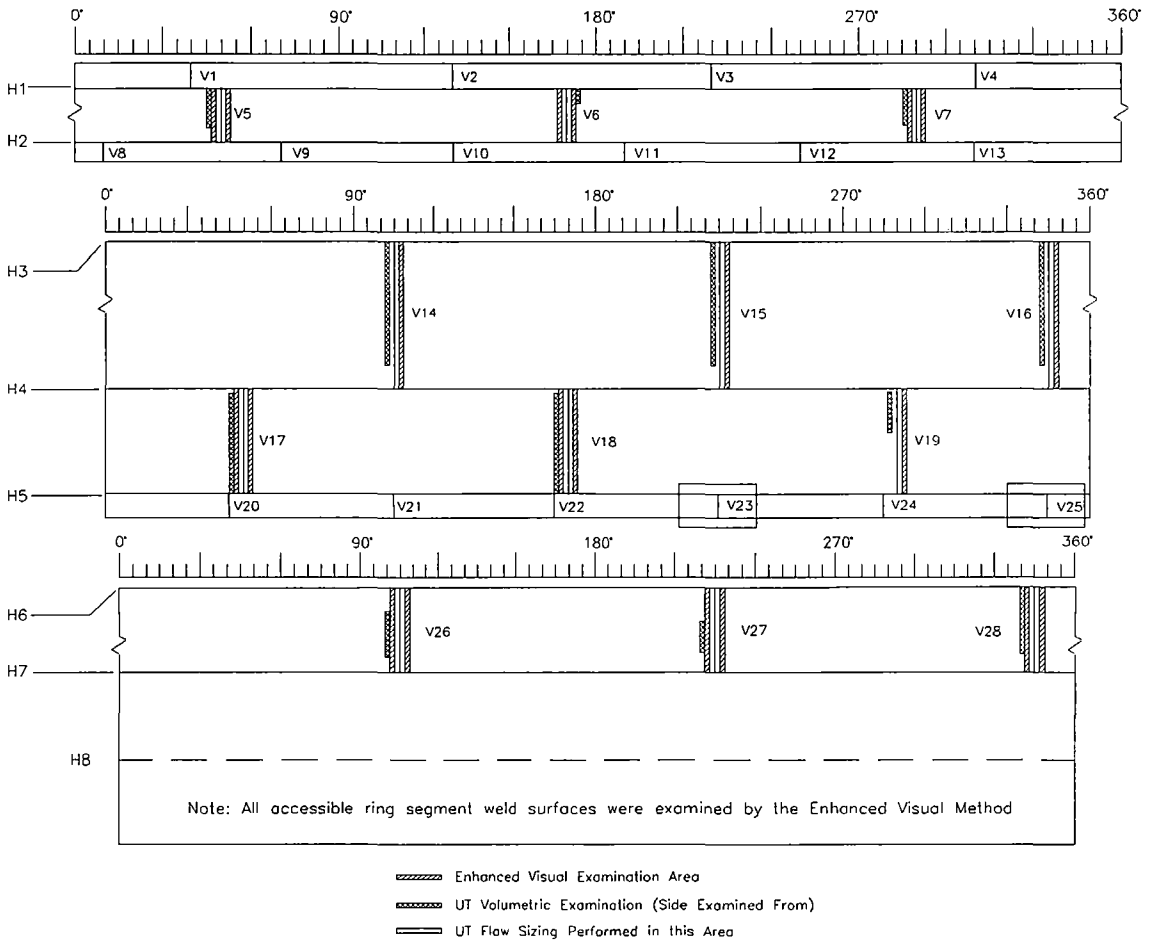
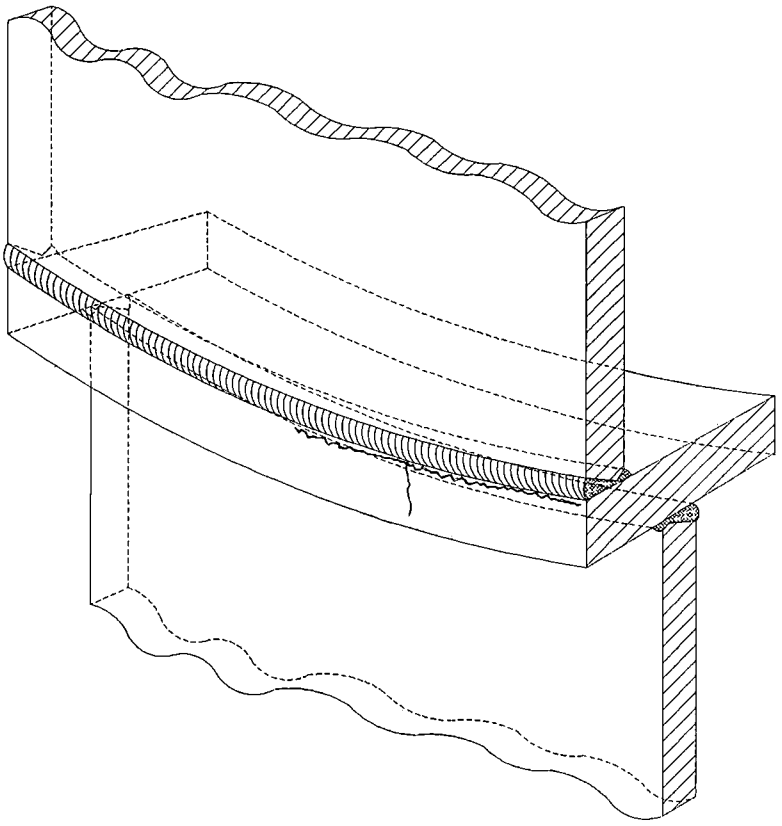


Figure 2

Core Plate Support Ring Indication (Typical)



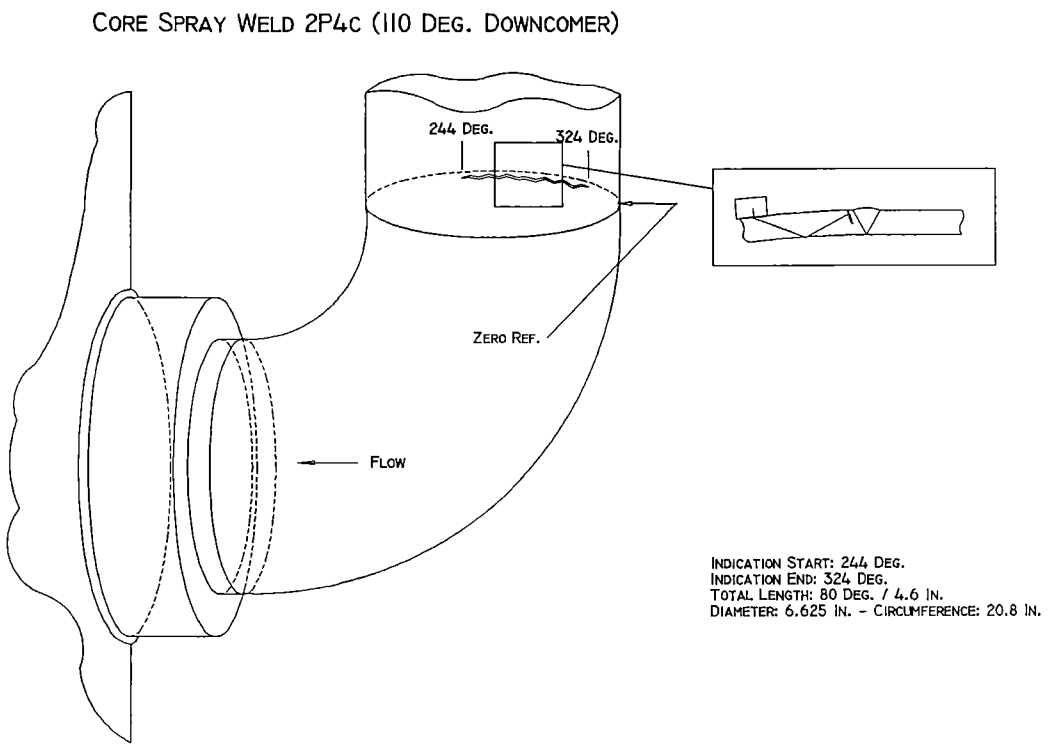


Figure 3
Flaw as located in the 2P4c Weld

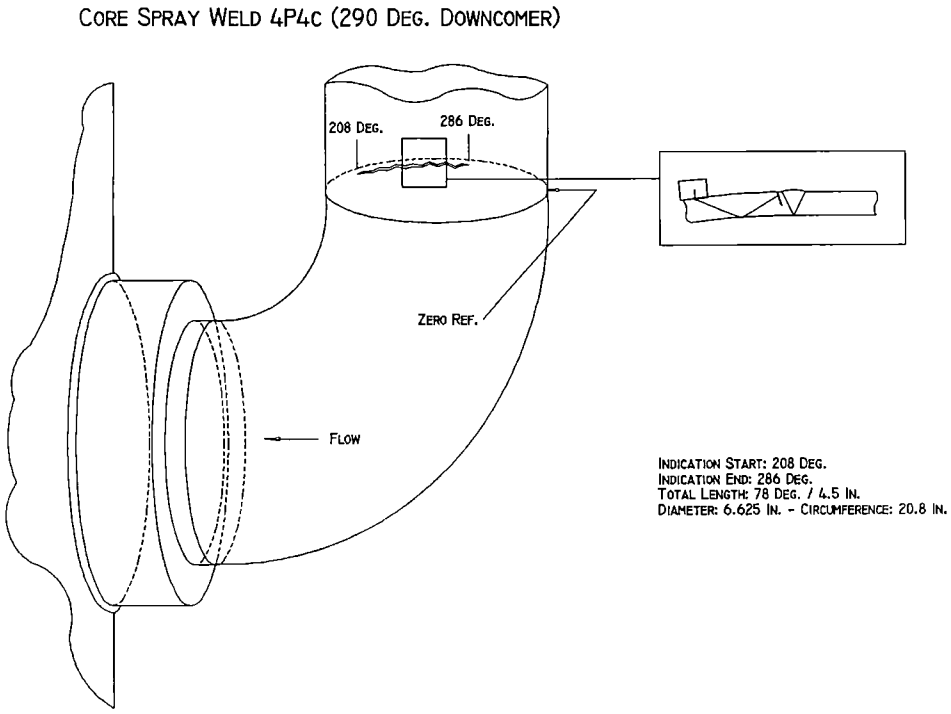


Figure 4
Flaw as located in the 4P4c Weld

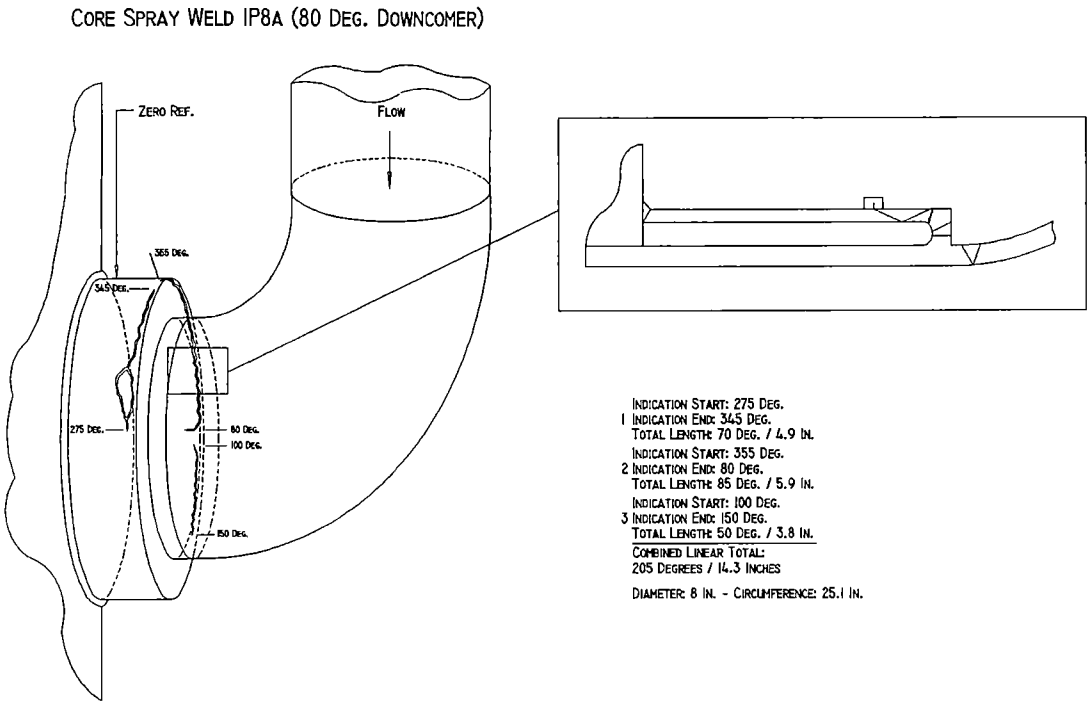


Figure 5
Flaw as located in the 1P8a Weld

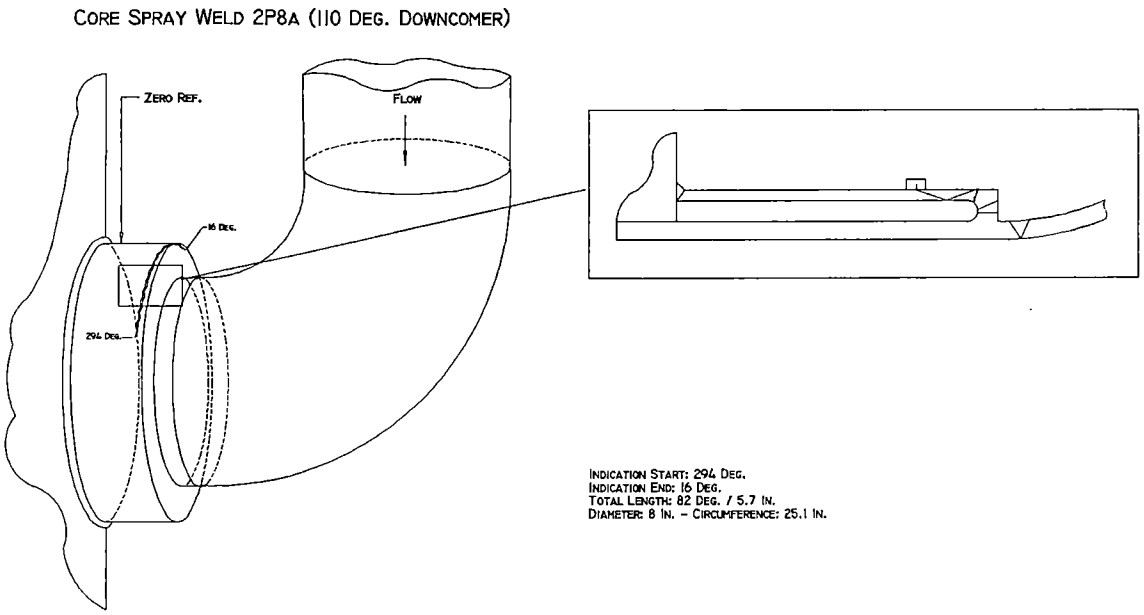


Figure 6
Flaw as located in the 2P8a Weld

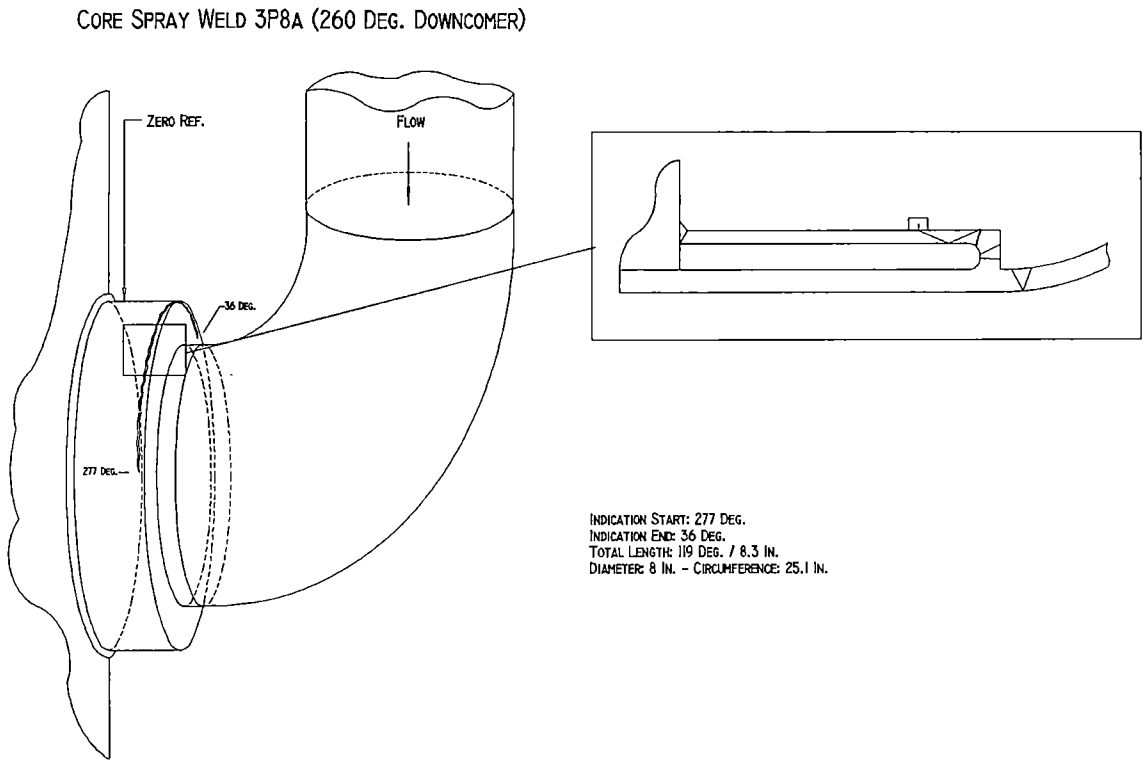
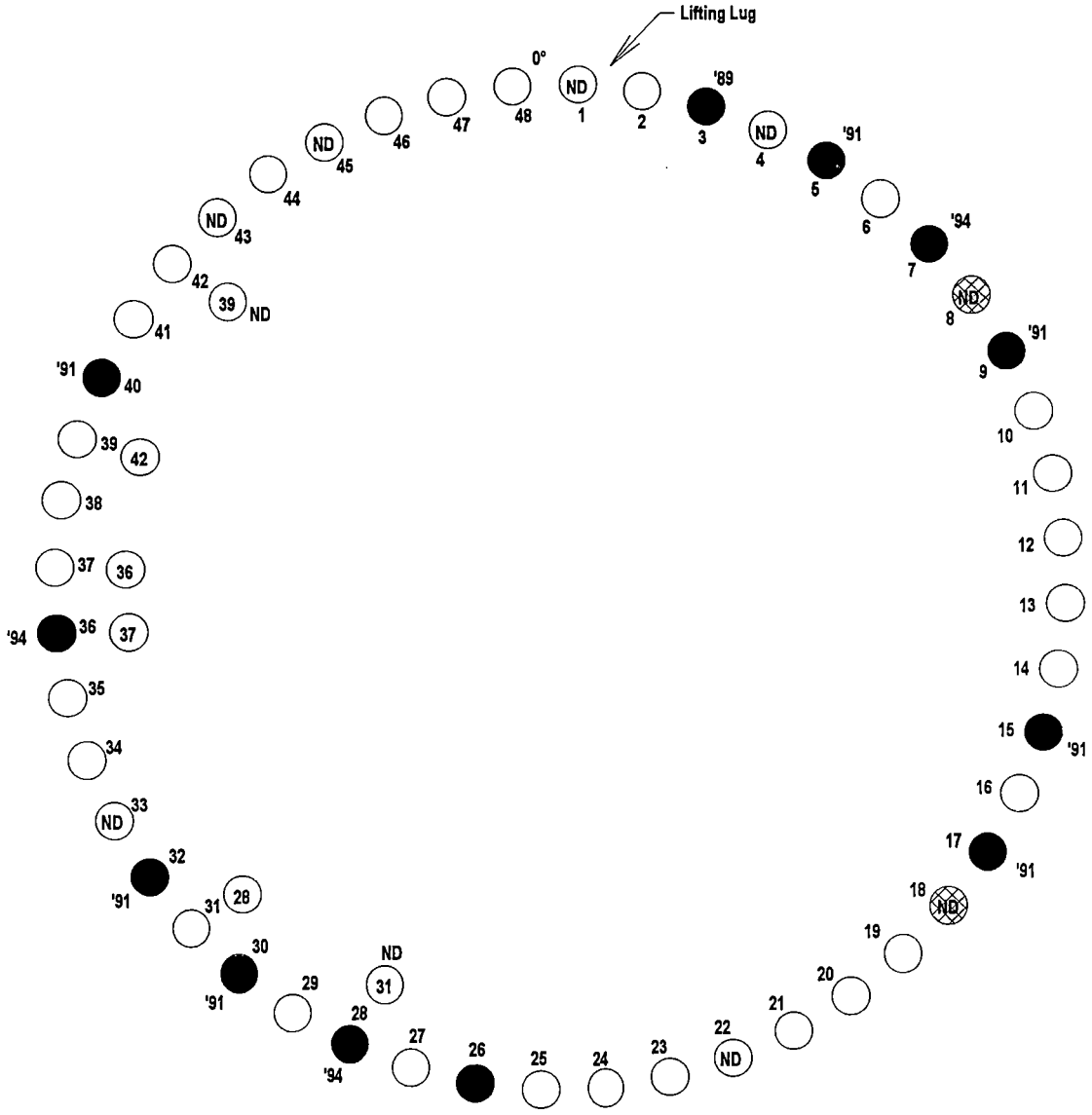


Figure 7
Flaw as located in the 3P8a Weld

Figure 8

D3R14 "As-Found" / "As-Left" Arrangement of Unit 3 Shroud Head Bolts



Commonwealth Edison Co.
P.O. Box 767, Chicago, IL 60690

Dresden Nuclear Power Station
6500 N. Dresden Road, Morris, IL 60450

Section III
Abstract Of Results, Evaluations, And Corrective Actions

The findings of the examinations and the corrective measures taken demonstrate that all components examined are functional and in compliance with the Dresden Unit 3 Technical Specifications and Section XI of the ASME Boiler and Pressure Vessel Code, 1989 Edition.

The following is a summary of corrective actions taken as a result of examination findings.

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Dresden Nuclear Power Station
6500 N. Dresden Road, Morris, IL 60450

Section III
Abstract Of Results, Evaluations, And Corrective Actions

Category	Item	Augment	System	Line	Component	Type
ISI and Augmented Examinations						
BA	B1.40		RPV	RPV UPP HD	3-THD-FLGD	THD-FLG

A magnetic particle examination of the "D" segment of the upper reactor head-to-flange weld revealed a linear indication exceeding Table IWB-3510-3 limits. A PIF was generated on 4/4/97. Systems Materials Analysis Department (SMAD) Metallurgy inspected the indication and determined it to be a lap. Indication was completely removed with light flapping (< 1/16" metal removed) under WR 960036553. The entire weld was examined, therefore no expansion was required.

BA	B1.40		RPV	RPV UPP HD	3-THD-FLGE	THD-FLG

A magnetic particle examination of the "E" segment of the upper reactor head-to-flange weld revealed two linear indications exceeding Table IWB-3510-3 limits. A PIF was generated on 4/4/97. SMAD Metallurgy inspected the indications and determined them to be laps. Indications were completely removed with light flapping (< 1/16" metal removed) under WR 960036553. The entire weld was examined, therefore no expansion was required.

BA	B1.40		RPV	RPV UPP HD	3-THD-FLGF	THD-FLG

A magnetic particle examination of the "F" segment of the upper reactor head-to-flange weld revealed two linear indications exceeding Table IWB-3510-3 limits. A PIF was generated on 4/4/97. SMAD Metallurgy determined the indications to be laps. The indications were completely removed with light flapping (< 1/16" metal removed) under WR 960036553. The entire weld was examined, therefore no expansion was required.

BP	B15.OT		N/A	TEST BLOCK	RCPB	N/A

A visual inspection (VT-2) in conjunction with the system leakage test of pressure testing block RCPB revealed leakage at the following components: Control Rod Drive (CRD) flanges A-6, G-8, J-5, L-6, and M-10; union/threaded connections at valves 3-0262-25B, 3-0262-26B, 3-0263-2-13A, 3-0263-2-17A, 3-0263-2-17B, and 3-0299-112B; flange on Line 3-0304-2.5" (reactor head spray piping); bonnets of valves 3-0203-2D, 3-0299-116B, 3-0305-101 (B-7), 3-0305-102 (A-7), 3-0305-126 (E-3), and 3-0305-127 (L-2); and CRD directional control valves 3-0305-120 (D-4) and 3-0305-120 (M-2).

The bolting at CRD flange G-8 was replaced during this outage, therefore no further inspection was required. Bolting at CRD flanges A-6, J-5, L-6, and M-10 was removed, examined visually (VT-3/4), and reinstalled under Work Request 950070273.

The unions/threaded connections at valves 3-0262-25B, 3-0262-26B, 3-0262-2-17A, 3-0263-2-17B, and 3-0299-112B were tightened with the system under pressure and inspected visually (VT-2) under Work Request 950070273. The union at valve 3-0263-2-13A was seal welded inspected visually (VT-2) under Work Request 960081352.

Per Relief Request PR-18, the bolting at flange 3-0304-2.5" and valves 3-0203-2D, 3-0299-116B, and 3-0305-127 (L-2) was retorqued with the system under pressure and reinspected visually (VT-2). The leakage was stopped, therefore no further actions were required.

Bonnet bolting for valves 3-0305-101 (B-7), 3-0305-102 (A-7), and 3-0305-126 (E-3) was removed, inspected visually (VT-3/4), found acceptable, and reinstalled under Work Request 950070273.

Leakage at CRD directional contol valves 3-0305-120 (D-4) and 3-0305-120 (M-2) was repaired under Work Requests 970063659 and 970063658.

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Section III
Abstract Of Results, Evaluations, And Corrective Actions

Category	Item	Augment	System	Line	Component	Type
ISI and Augmented Examinations						
CH	C7.OT		N/A	TEST BLOCK	03A1	N/A

A visual inspection (VT-2) in conjunction with the system leakage test of pressure testing block 03A1 revealed leakage at the bonnet of Valve 3-0305-112 (C-5).

All four bolts were removed and visually inspected (VT-3/4) under Work Request 950070273. One bolt was identified as being bent and one bolt had pitting located at the threads of the bolt. Both bolts were replaced with new bolts and the two remaining bolts (which were acceptable) were reinstalled.

FA	F1.30		CCSWAD	1514D-8	M-1200D-259	CL 3 SUP

A visual examination of CCSWAD support M-1200D-259 (support at discharge of 3A CCSW pump) revealed a discrepancy between the as-built configuration of the support and the drawing. The drawing shows a weephole which does not exist on the actual support. The remaining CCSW pump supports (3B, 3C and 3D) were examined and had the same drawing discrepancy. A PIF was initiated on 4/1/97 and Engineering Request 9701343 was initiated on 4/2/97. The discrepancies are not service induced, therefore no expansion was required.

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Section IV
Abbreviations

Component Type

BPC	Branch Pipe Connection
BPCS	Branch Pipe Connection Saddle
CAP	Pipe Cap
COND	Condenser
CRO	Cross
EL	Elbow
ELS	Elbow Longitudinal Seam
F	Fuel Head
FLG	Flange
FLGBLT	Flange Bolt
FLS	Fitting Longitudinal Seam
HTEX	Heat Exchanger
IWA	Integral Welded Attachment
NIR	Nozzle Inner Radius
NOZ	Nozzle
P	Pipe
PG	Penetration Guide
PLS	Piping Longitudinal Seam
PMP	Pump
PMPBLT	Pump Bolting
RED	Reducer
REDE	Reducing Elbow
RPV	Reactor Pressure Vessel
SDL	Saddle
SE	Safe-end
SHL	Shell
SWC	Socket Welded Coupling
SWCP	Socket Welded Pipe Cap
SWE	Socket Welded Elbow
SWF	Socket Welded Flange
SWP	Sweep-O-Let, Weld-O-Let, Etc.
SWR	Socket Welded Reducer
SWT	Socket Welded Tee
SWV	Socket Welded Valve
TBSHT	Tubesheet
TEE	Tee
VB	Vacuum Breaker
VLV	Valve
VLVBLT	Valve Bolting

Section IV
Abbreviations

Credit

06	NUREG 0619
88	Generic Letter 88-01
OR	Other Special Exam to be explained in memo field
XI	Section XI

Other

DR	Discrepancy Record
PIF	Performance Improvement Form

Exam

FT	Functional Test
MT	Magnetic Particle
PT	Liquid Penetrant
UT	Ultrasonic
VT-1	VT-1 visual
VT-2	VT-2 visual
VT-3/4	VT-3/4 visual

System

CCSWAD	Containment Cooling Service Water "A", Pump Discharge
CRD	Control Rod Drive
CRDH	Control Rod Drive, Hydraulic
CRDSD	Control Rod Drive, Scram Discharge Volume
CSAD	Core Spray "A", Pump Discharge
CSAS	Core Spray "A", Pump Suction
CSBD	Core Spray "B", Pump Discharge
CSBS	Core Spray "B", Pump Suction
DGSW	Diesel Generator Service Water
ECCS	Emergency Core Cooling System Ring Header
FW2	Feedwater, Class 2
FWA	Feedwater "A"
FWB	Feedwater "B"
HPCIPD	High Pressure Coolant Injection, Pump Discharge
HPCIPS	High Pressure Coolant Injection, Pump Suction

Section IV
Abbreviations

HPCISS	High Pressure Coolant Injection, Steam Turbine Supply
HPCITE	High Pressure Coolant Injection, Turbine Exhaust
ISCOCR	Isolation Condenser, Condensate Return
ISCOSS	Isolation Condenser, Steam Supply
ISCOVP	Isolation Condenser and Vent Piping
JPIA	Jet Pump Instrumentation Loop "A"
JPIB	Jet Pump Instrumentation Loop "B"
LPCIAD	Low Pressure Coolant Injection "A", Pump Discharge
LPCIAS	Low Pressure Coolant Injection "A", Pump Suction
LPCIBD	Low Pressure Coolant Injection "B", Pump Discharge
LPCIBS	Low Pressure Coolant Injection "B", Pump Suction
LPCIHX	Low Pressure Coolant Injection Heat Exchengers
LPCISR	Low Pressure Coolant Injection Torus Spray Ring
LPCITR	Low Pressure Coolant Injection Test Return to Torus
LPCIX	Low Pressure Coolant Injection Crosstie
LVLA	Lower Vessel Level "A"
LVLB	Lower Vessel Level "B"
MSA	Main Steam "A"
MSB	Main Steam "B"
MSC	Main Steam "C"
MSD	Main Steam "D"
MSDN	Main Steam Drain
RHS	Reactor Head Spray
RHV	Reactor Head Vent
RPV	Reactor Pressure Vessel
RRAD	Reactor Recirculation Loop "A", Pump Discharge (U/2 includes the crosstie piping up to but not including weld 202-6B/L3)
RRAS	Reactor Recirculation Loop "A", Pump Suction
RRBD	Reactor Recirculation Loop "B", Pump Discharge (U/2 includes the crosstie piping up to but not including weld 202-6B/L3)
RRBS	Reactor Recirculation Loop "B", Pump Suction
RVBD	Reactor Vessel Bottom Drain
RWCU	Reactor Water Clean Up
SBLC	Standby Liquid Control
SDC	Shutdown Cooling
SRVDA	Safety Relief Valve Discharge "A"
SRVDB	Safety Relief Valve Discharge "B"
SRVDC	Safety Relief Valve Discharge "C"
SRVDD	Safety Relief Valve Discharge "D"
SRVDE	Safety Relief Valve Discharge "E"
UVLA	Upper Vessel Level "A"
UVLB	Upper Vessel Level "A"

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Section V

Examinations, Tests, Replacements, And Repairs Since The Preceding Summary Report

Several ASME Section XI repairs and replacements have taken place at Dresden Unit 3 since the previous summary report was issued. A review of the Dresden Station Section XI Repair Program Log was conducted in order to identify the various repairs and replacements.

Copies of the NIS-2 forms associated with all of the Section XI repairs and replacements conducted since the previous summary report have been included in this section. The NIS-2 forms provide an abstract of the repairs and replacements and outline the examinations and tests performed in conjunction with them.

A listing of NIS-2 forms is included in this section in order of repair/replacement plan number followed by the work request number.

Section V

Examinations, Tests, Replacements, And Repairs Since The Preceding Summary Report

NIS-2 No.	Work Request
3-93-014	930054156
3-93-034	930051379
3-94-001	920052847
3-94-002	930054362
3-94-003	930054363
3-94-004	930054364
3-94-005	930054365
3-94-006	930054366
3-94-007	930054367
3-94-008	930054368
3-94-009	930054369
3-94-010	930054370
3-94-011	930054371
3-94-014	930054374
3-94-017	930054451
3-94-026	930054528
3-94-027	930054529
3-94-030	930054527
3-94-037	930054424
3-94-041	930054470
3-94-042	930054469
3-94-043	930054468
3-94-046	940093367
3-94-047	940093368
3-94-048	940093385
3-94-049	940093348
3-94-052	920052857
3-94-053	920052858
3-94-055	920057892
3-94-056	920051524
3-94-057	920051521
3-94-058	910056669
3-94-061	910056672
3-94-065	910051255
3-94-066	920051827
3-94-067	920051828
3-94-069	940093535
3-94-070	940094760
3-94-071	930053406
3-94-072	930053701
3-94-073	930056992
3-94-076	930053997
3-94-094	920055929

Section V

Examinations, Tests, Replacements, And Repairs Since The Preceding Summary Report

NIS-2 No.	Work Request
3-94-097	930054138
3-94-098	940095674
3-94-103	930053501
3-94-107	940096138
3-94-112	940093916
3-94-115	940096775
3-94-116	940096996
3-94-117	940097520
3-94-119	940097957
3-95-006	930053598
3-95-007	930053599
3-95-008	950065745
3-95-009	950060470
3-95-010	950061024
3-95-011	950060471
3-95-012	950061025
3-95-013	950060465
3-95-014	950060469
3-95-015	950060661
3-95-016	940095273
3-95-017	950065747
3-95-018	950069482
3-96-009	940096861
3-96-011	950063282
3-96-012	950063489
3-96-015	950046326
3-96-017	950047139
3-96-019	950097245
3-96-020	960049833
3-96-021	930053801
3-96-022	930053800
3-96-023	960030678
3-96-024	960030680
3-96-026	960057668
3-96-028	960030678
3-96-030	960049569
3-96-031	960049569
3-96-032	950063471
3-96-033	960030678
3-96-034	960081175
3-96-038	950063469
3-97-001	960011774
3-97-002	960118198

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Dresden Nuclear Power Station
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Section V

Examinations, Tests, Replacements, And Repairs Since The Preceding Summary Report

NIS-2 No.	Work Request
3-97-003	960118148
3-97-004	960011774
3-97-006	960096685
3-97-007	950063467
3-97-008	950063481
3-97-009	940096467
3-97-010	960116198
3-97-011	970017094
3-97-012	970017094
3-97-013	970013877
3-97-014	970031638
3-97-017	940096861
3-97-018	960036553
3-97-020	970044798
3-97-021	950063479
3-97-022	950063478
3-97-025	950061006
3-97-027	950060516
3-97-029	970048050
3-97-030	970052950
3-97-031	960001532
3-97-033	970052619
3-97-035	960032654
3-97-036	960032651
3-97-037	970057096
3-97-038	950070767
3-97-039	950065053

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 4/15/94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: COMMONWEALTH EDISON (Name)
SAME (Address)

WR. D20576 [illegible] 793-04
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 6600

5. (a) Construction Code TEMA SARGENT + LUNDY SPEC K-2380, 19 — Edition, NONE Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NONE Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
D/G COOLING WTR HT. EX	YOUNG RADIATOR CO	346544	N/A	3-6669A	N/A	REPLACED	NO
D/G COOLING WTR HT EX	YOUNG RADIATOR CO	346545	N/A	3-6669B	N/A	REPLACED	NO
D/G COOLING WTR HT EX	YOUNG RADIATOR CO	2036847	N/A	3-6669A	N/A	REPLACEMENT	NO
D/G COOLING WTR HT EX	YOUNG RADIATOR CO	825854	N/A	3-6669B	N/A	REPLACEMENT	NO

7. Description of work: REMOVED EXISTING HT. EXCHS. AND INSTALLED NEW

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 22 psig Test Temperature 58 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 1-8 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&I CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 1-8-96 19__ (BJC 1-11-96) and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-8-96 (BJC 1-11-96) Inspector: [signature] Commissions: IL 932 NB 7742 NISB (State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 4-22-96

Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: SAME AS ABOVE (Name)

SAME AS ABOVE (Address)

D17980 PLAN 3-93-034
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3000 MAIN STEAM

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" GLOBE VALVE	CRANE	N/A	N/A	MODEL 7852-U	N/A	REPLACED	NO
2" SEAMLESS PIPE	UNKNOWN	N/A	N/A	NONE	N/A	REPLACED	NO
1½" SOCKET WELD ELBOWS	UNKNOWN	N/A	N/A	NONE	N/A	REPLACED	NO
1½" SEAMLESS PIPE	UNKNOWN	N/A	N/A	NONE	N/A	REPLACED	NO
2" DOUBLE DISC GATE VALVE	ANCHOR DARLING	ET852-5-1	N/A	SI #812D48	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing globe valve with double disc gate valve to address Generic Letter 89-10 issues and reconfigured piping and supports under Exempt Plant Change P12-3-94-203.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 4-23 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on 4-29, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 4-29-96 Inspector: Rick T Kairney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

ATTACHMENT 1

NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: ComEd
One First National Plaza, Chicago IL, 60690

Date: 4-26-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Staion
6500 N. Dresden Road, Morris IL., 60450

Unit: 3

3. Work Performed By: BECHTEL CONSTRUCTORS (Name)

GAITHERSBURG, MARYLAND (Address)

D09555/920052847 PLAN 3-94-001
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1300 ISOLATION CONDENSER

5. (a) Construction Code USAS B31.1.0, 1967 Edition, NO Addenda, Code Cases NO
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NO

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
14" A358 Grade 304 PIPE	UNKNOWN	N/A	N/A	LINE 3-1302-14"-A	N/A	REPLACED	NO
14" A358 Grade 304 TYPE L PIPE	YOUNGSTOWN WELDING & ENGINEERING	HT# 2A804	N/A	LINE 3-1302-14"-A	1993	REPLACEMENT	NO

7. Description of work: Cut out existing pipe spool piece in order to install corrosion resistant cladding at inaccessible circumferential welds. Install new pipe spool piece after completion of weld inlay.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: Installed corrosion resistant cladding in piping located inside of flued head per Modification M12-3-91-002.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 8-19 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 8-21, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 8-21-95 Inspector: Kurt T Rasmussen Commissions: IL932, NB7742NISB
(State or Province, National Board)

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 2-9-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D20650 (3-94-002)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	641C	N/A	SN 641C	'67	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9495	N/A	SN A9495	'75	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	Unk.	N/A	1-8 x 5½"	Unk.	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT 52613	N/A	1-8 x 5½"	Unk	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure ~~1100~~ 1040 psig BAL 11/10/94 Test Temperature ~~180~~ 200 °F BAL 11/10/94

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION 35-23 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR UT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB H&I CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 3-16, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-16-95 Inspector: [signature] Commissions: 16932, NB7742 N I S B
(State or Province, National Board)

RIN/QRI M94-00219

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

   2117 Castle Hayne Road, Wilmington, North Carolina 28401
   (Name and Address of NPT Certificate Holder)

   (b) Manufactured for : Dresden Morris, Illinios 60450
   (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9495 Nat'l Bd. No. N/A

   (a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

   (b) Description of Part Inspected: Control Rod Drive , Model # 7RDB144FG001

   (c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
   (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (SC QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company , San Jose , California

Stress analysis report on file at GE Company , San Jose , California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 2/11, 1994, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 Jerome P Evans NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

As Required by [illegible] of ASME Code Section XI

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 2-9-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20651 (3-94-003)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	6539	N/A	SN 6539	74	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9538	N/A	SN A9538	93	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT52613	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION P-9 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C CO of HARTFORD, CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-14-, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: [signature] Commissions: IL 432, NB 7242 N IS 3
(State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

   2117 Castle Hayne Road, Wilmington, North Carolina 28401
   (Name and Address of NPT Certificate Holder)

   (b) Manufactured for : Dresden Morris, Illinios 60450
   (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : [redacted] Nat'l Bd. No. N/A

   (a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

   (b) Description of Part Inspected: Control Rod Drive , Model # 7RDB144FG001

   (c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
   (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By [signature]
(NPT Certificate Holder) (SC OR Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company , San Jose , California

Stress analysis report on file at GE Company , San Jose , California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 11/2, 1993, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 [signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20652 (3-94-004)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	973	N/A	SIN 973	69	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9502	N/A	SIN A9502	93	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT#52613	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION X-[illegible] AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR UT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&CO of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-14, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: [signature] Commissions: IL932, NB7742 N I S B (State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

   2117 Castle Hayne Road, Wilmington, North Carolina 28401
   (Name and Address of NPT Certificate Holder)

   (b) Manufactured for : Dresden Morris, Illinios 60450
   (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9502 Nat'l Bd. No. N/A

   (a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

   (b) Description of Part Inspected: Control Rod Drive , Model # 7RDB144FG001

   (c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
   (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (SC QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company , San Jose , California

Stress analysis report on file at GE Company , San Jose , California

DC22A6253 Rev. 1
Design specification certified by Biorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 2/11, 1994, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 Jerome P Emery NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

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FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7-15-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Owner (Name)
Same (Address)

D20653 (3-94-005)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 Control Rod Drive

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases None

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases None

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive	General Electric	93	N/A	3-0300-C11	68	Replaced	Yes
Control Rod Drive	General Electric	259	N/A	3-0300-C11	68	Replacement	Yes
CRD Flange Bolts (8)	General Electric	None	N/A	None	N/A	Replaced	NO
CRD Flange Bolts (8)	General Electric	HT# 52613	N/A	None	N/A	Replacement	NO

7. Description of work: Remove existing Control Rod Drive assembly and flange bolting (capscrews) and replace with rebuilt CRD and new flange bolts.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: Removed existing Control Rod Drive and flange bolts from reactor position C11 and replaced with rebuilt CRD and flange bolts.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+CO of HARTFORD CT having inspected the Replacement (Repair or Replacement) described in this report on 3-14, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: [signature] Commissions: IL 932, NB 7742 N I S B (State or Province, National Board)

01607

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer

As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 259, 320

NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date February 16, 19 68 Signed General Electric Co, APED (Manufacturer) By [signature]

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. ____ State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng ____ State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on ________ 19 ____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2-14- 19 68

[signature] Commissions C-706
Inspector's Signature National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20654 (3-94-006)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	884A	N/A	SN 884A	69	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	684C	N/A	SN 684C	67	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT 52613	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure ~~1000~~ 1040 psig RJS 6/24/94 Test Temperature ~~180~~ 200 °F RJS 6/24/94

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION C-3 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-14-95, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: [signature] Commissions: IL 932, NB 7742 NISB (State or Province, National Board)

01491

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer

As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102: 333-C,426-C,454-C,467-C,553-C,586-C,607-C,608-C,611-C, 684-C,721-C,756-C,773-C , 516-C, 720-C

(a) Constructed According to Drawing No. 237E179G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases No. 1335-2, 1361, and 1352.

See sketch showing configuration and materials used. Hydrotested at 2110 psi.

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date September 29 19 67 Signed General Electric Co., APED By [signature]
(Manufacturer)

Certificate of Authorization Expires Dec 31, 1967

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept of Industrial Relaitons have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19_____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 9-29- 19 67

[signature]
Inspector's Signature

Commissions 13036
National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20655 RFP3-94-007
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS.300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	708C	N/A	S/N 708C	67	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	275	N/A	S/N 275	67	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5 1/2"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT#52613	N/A	1-8 x 5 1/2"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION K-9 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS. WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C Co of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 9-14, 19 95 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: [signature] Commissions: IC932, NB7742 N&B (State or Province, National Board)

01582

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co. APED 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102, 74, 188, 275

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date December 28, 1967 Signed General Electric Co, APED (Manufacturer) By [signature]

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____, 19____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-2-68 19 67

[signature]
Inspector's Signature

Commissions [illegible]
National Board or State and No

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20656 (3-94-008)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	283	N/A	SN 283	65	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A8504	N/A	SN A8504	65	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	CODE H.T. LOG	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]
Test Pressure ~~1400~~ 1040 RJS 8/24/94 psig Test Temperature ~~150~~ 200 RJS 8/24/94 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION J-7 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS. WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C Co of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-14-, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: [signature] Commissions: IL932 NB7742 NISB (State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by: GE Company, 2117 Castle Hayne Rd., Wilmington, N.C. 28402
(Name and Address of NPT Certificate Holder)
(b) Manufactured for: DRESDEN -2, MORRIS, ILLINOIS 60450
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification-Certificate Holders's S/N of Part: A8504 Nat'l Bd. No. N/A
(a) Constructed According to Drawing No: 919D258G003 Dwg. Prepared by D. L. Peterson
(b) Description of Part Inspected: CYLINDER TUBE & FLANGE
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75 Case No. 1361-2 Class 1

3. REMARKS: Sub-assembly of Control Rod Drive for use with reactor.
(Brief description of service for which component was designed)
Hydrostatically tested at 1825 psi. min.

*Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

DATE: 3/11, 19 88 Signed GE-NEBG-NF&CM-QA By [signature]
(NPT Certificate Holder)

Certificate Of Authorization Expires: 6/16/90 Certification of Authorization No.: NPT N-1151

CERTIFICATION OF DESIGN FOR APPURTENANCE

Design information on file at GE COMPANY, SAN JOSE, CALIFORNIA

Stress analysis report on file at GE COMPANY, SAN JOSE, CALIFORNIA
DC22A6253 Rev. 0
Design specification certified by BJORN HAABERG Prof. Eng. State CALIF. Reg. No. 15570
DC22A6254 Rev. 0.
Stress analysis report certified by EDWARD YOSHIO Prof. Eng. State CALIF. Reg. No. M018646

CERTIFICATION OF SHOP INSPECTION

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of NORTH CAROLINA and employed by DEPARTMENT OF LABOR of STATE OF NORTH CAROLINA have inspected the part of a pressure vessel described in this Partial Data Report on 3/15 19 88, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

3/15, 19 88 [signature] N.C. 723 [illegible]
DATE Inspector's Signature National Board, State, Province and No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" X 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKSs"

10/77)

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D20697 (3-94-009)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	600C	N/A	S/N 600C	67	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	959	N/A	S/N 959	69	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT#52613	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION H-3 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+I CO of HARTFORD, CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 7-14-, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 7-14-95 Inspector: [signature] Commissions: IL932, NB7742 H IS B (State or Province, National Board)

01724

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 806, 876, 878, 887, 930, 946, 959, 995
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date January 24, 19 69 Signed General Electric Co, APED By
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11158

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13560

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19_____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24- 19 69

M. A Dawes Commissions [illegible]
Inspector's Signature National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20658 (3-94-010)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS.300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	613C	N/A	S/N 613C	67	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	537C	N/A	S/N 537C	67	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT#52613	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION G8 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&I CO of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-14-, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: [signature] Commissions: IL932, NB7742 N,I,S,B (State or Province, National Board)

01624

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102; 390-C, 509-C, 537-C, 588-C, 688-C, 753-C, 772-C

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date November 8 19 67 Signed General Electric Co. APED By G. W. Zinke
(Manufacturer)

Certificate of Authorization Expires December 31, 1967

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. ____ State Calif Reg. No M11138

Stress analysis report certified by R. L. Call Prof. Eng. ____ State Calif Reg. No 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrail Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11-8- 1967

Inspector's Signature Commissions ____
National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D20659 (3-94-011)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	7328	N/A	S/N 7328	78	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9546	N/A	S/N A9546	93	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HH52613	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTRO
ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION R-
AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR
VT-1 INSPECTED BOLTS

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+CO of HARTFORD, CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-14-, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-14-95 Inspector: R Kaney Commissions: IL932, NB7742 N1513 (State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PARTS AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

   2117 Castle Hayne Road, Wilmington, North Carolina 28401
   (Name and Address of NPT Certificate Holder)

   (b) Manufactured for : Dresden Morris, Illinios 60450
   (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9546 Nat'l Bd. No. N/A

   (a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

   (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG001

   (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W'75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
   (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (ISC QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 11/2, 1993, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 Jerome P Ewers NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/14/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20662 (3-94-014)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS.300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	A5021	N/A	S/N A5021	82	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	978	N/A	S/N 978	69	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HH052613	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION F-11 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&I CO of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 7-30-94, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-30-94 Inspector: [signature] Commissions: IL 932 NB 7142 N IS B
(State or Province, National Board)

01829

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT
A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 978, 981, 1039, 1056
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date February 18, 19 69 Signed General Electric Co, R&FMO By
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No M1113

Stress analysis report certified by R. L. Call Prof. Eng M.E. State Calif Reg. No 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19_____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2-19- 19 69

M A Dawe
Inspector's Signature

Commissions Cal 706
National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL. 60690 (Address)

Date: 7/16/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

P20728 (Plan 3-94-017)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	1775	N/A	SN 1775	70	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9674	N/A	SN A9674	65	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT CODE KOG	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure ~~1100~~ 1040 RJS 8/24/94 psig Test Temperature ~~150~~ 200 RJS 8/24/94 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION M-1[illegible] AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 12-5 (Date), 19 95

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+I CO of HARTFORD, CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 12-21, 19 95 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-29-95 Inspector: [signature] Commissions: IL 1092 NB 7747 N I S B (State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PARTS AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : Dresden Morris, Illinois 60450
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9674 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive , Model # 7RDB144FG001

(c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

RIN/QRI M94-00719

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (SC QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Biorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 9/15, 1993, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 Date — Inspector's Signature — NC 1231, Ohio, WC 3686 PA — National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL 60690 (Address)

Date: 7/19/93

Sheet: 1 of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20836 (3-94-026)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: A-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, No Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	498C	N/A	S/N 498C	67	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	929	N/A	S/N 929	65	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	WITH KOG	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1090 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION L-4 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C of HARTFORD, CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 7-30, 19 95 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: K932, IL 1742, NB

(State or Province, National Board)

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 782, 929, 934, 967, 979, 1032, 1040

NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date January 31, 19 69 Signed General Electric Co. APED (Manufacturer) By (signature)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11158

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13510

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on 1-31 19 69, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-31 19 69

(signature)
Inspector's Signature

Commissions Cal 849
National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/19/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20837 (3-94-027)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS.300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	806	N/A	SN 806	69	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	8	N/A	SN 8	65	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 X 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT CODE KOG	N/A	1-8 X 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]
1040 RVS 8/25/94 200 RVS 8/25/94
Test Pressure ~~1100~~ psig Test Temperature ~~150~~ °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION M-5 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB IT ICU of HARTFORD CT. having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-30, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: 16932, NB7742 NI&B (State or Province, National Board)

01589

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102: 8, 51, 78, 106, 107, 134

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date January 15, 19 68 Signed General Electric Co, APED By W. R. Perrault
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on ________ 19____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-16 19 68

[signature] Commissions Cal 706
Inspector's Signature National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/19/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20835 (3-94-030)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS.300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	7455	N/A	SN 7455	78	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	679C	N/A	SN 679C	67	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 X 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT 52013	N/A	1-8 X 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]
Test Pressure 1040 psig Test Temperature 150 °F
1040 RJB 8/25/94 200 RJS 8/25/94

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION D-9 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-30-95, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: IL932, NB7742, N1513 (State or Province, National Board)

01516

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102: 587-C, 605-C, 660-C, 679-C, 710-C, 761-C, 735-C,

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date October 20, 19 67 Signed General Electric Co, APED (Manufacturer) by [signature]

Certificate of Authorization Expires December 31, 1967

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design Specifications certified by W. Schultheis Prof. Eng. State Calif Reg. No M11 38

Stress analysis report certified by R. L. Call Prof. Eng. State Calif Reg. No 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19___, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 10-20 19 67

[signature]
Inspector's Signature

Commissions [signature]
National Board or State and No.

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/15/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20755 (Plan 3-94-037)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, No Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	# 1	N/A	SN # 1	67	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9545	N/A	SN A9545	74	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	CODE HT KMA	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic (X) Pneumatic ( ) Nominal Operating Pressure ( ) Not Applicable ( )
Test Pressure ~~1100~~ 1040 RJD 8/25/94 psig Test Temperature ~~180~~ 200 RJD 8/25/94 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION E-4 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 12-5 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+I CO of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 12-29, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-29-95 Inspector: [signature] Commissions: IL932 NB7742NIS (State or Province, National Board)

RIN/QRI M94-00719

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Division 1

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : Dresden Morris, Illinios 60450
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9545 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG001

(c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (GE QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Biorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 12/2, 1993, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

11/2, 1994 — Date
Inspector's Signature
NC 1231, Ohio, WC 3686 PA — National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

D94-00656 (07/00)

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/16/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20841 (3-94-041)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	A5444	N/A	S/N A5444	81	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9630	N/A	S/N A9630	83	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 X 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HTHS2001	N/A	1-8 X 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION B6 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS. WITH NEW OR VT-1 INSPECTED BOLTS

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C Co of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-30, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: IL932, NR7742 N1SB (State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : Dresden Morris, Illinios 60450
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9630 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G085 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG001

(c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (SC QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Biorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 11/5, 1993, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 Inspector's Signature NC 1231, Ohio, WC 3686 PA
Date National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

As Required by the Provisions of ASME Code Section XI

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL 60690 (Address)

Date: 7/16/93

Sheet: 1 of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20840 (3-94-042)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	78	N/A	S/N 78	68	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9339	N/A	S/N A9339	7A	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HTH KOG	N/A	1-8 x 5½"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION K-4 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+CO of HARTFORD, CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 3-30-, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: 14932, NB7742 NISB (State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)

   2117 Castle Hayne Road, Wilmington, North Carolina 28401
   (Name and Address of NPT Certificate Holder)

   (b) Manufactured for : Dresden Morris, Illinios 60450
   (Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9339 Nat'l Bd. No. N/A

   (a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

   (b) Description of Part Inspected: Control Rod Drive , Model # 7RDB144FG001

   (c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
   (Brief description of service for which component was designed)

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (GE QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 2/11, 1994, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 Jerome P Evans NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7/16/93

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 20839 (3-94-043)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	599	N/A	SN 599	68	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	977	N/A	SN 977	69	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5 1/2"	UNK	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT 52613	N/A	1-8 x 5 1/2"	UNK	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]
1040 RJS 8/25/94 200 RJS 8/25/94
Test Pressure ~~1020~~ psig Test Temperature ~~150~~ °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION H12 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C CO of HARTFORD CT having inspected the REPLACEMENT (Repair or Replacement) described in this report on 7-30, 1994 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: R Mauney Commissions: IL 932, NB 7742 N IS B (State or Province, National Board)

01874

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 888A, 900, 906, 977, 1030, 1046, 1049, 1052, 1057, 1062, 1070, 1080, 1085
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date March 5, 19 69 Signed General Electric Co, R&FMO by
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-5- 19 69

Inspector's Signature Commissions National Board, State, and/or

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 11-10-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 23930 (3-94-046)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 19 65 Edition, N° Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, N6 Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	1024	N/A	S/N 1024	69	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A4136	N/A	S/N A4136	65	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 x 5½"	N/A	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HB52613	N/A	1-8 x 5½"	N/A	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]
Test Pressure 1040 1000 psig BAC 11/10/94 Test Temperature 200 100 °F BAC 11/10/94

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION D-13 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 3-30-95, 19 ___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: R Clancy Commissions: IL 932, NB 7742, NIS B
(State or Province, National Board)

FORM N-3 NPT CERTIFICATE HOLDER'S DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*

As required by the Provision of the ASME Code Rules, Section III, Div. 1

CORRECTED COPY

1. (a) Manufactured by General Electric Company, Castle Hayne Rd., Wilmington, N.C.
(Name and address of NPT Certificate Holder)

(b) Manufactured for DRESDEN 2/3
(Name and address of N Certificate Holder for completed nuclear component)

2. Identification-Certificate Holder's Serial No. of Part A4136 Nat'l Bd. No.

(a) Constructed According to Drawing No. 768E534G005 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive, Model #7RDB144FG001

(c) Applicable ASME Code: Section III, Edition 1974, Addenda date W'75, Case No. N207 1361-2 Class 1

3. Remarks: Standard part for use with Reactor. Hydrostatically tested at 1825 psi.
(Brief description of service for which component was designed)

* Total Number of sheets-2

CORRECTED COPY: CHANGED CERTIFICATION OF DESIGN FOR APPURTENANCE SPECIFICATION.

We certify that the statements made in this report are correct and this vessel part or appurtenance as defined in the Code conforms to the rules of construction of the ASME Code Section III.
(The applicable Design Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certificate Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date 4/17 19 87 Signed GE-NEBG-NF&CM-QA By J. Ettorre
(NPT Certificate Holder)

Certificate of Authorization Expires June 16, 1997 Certificate of Authorization No. NPT N-1151

CERTIFICATION OF DESIGN FOR APPURTENANCE (when applicable)

Design information on file at GENERAL ELECTRIC CO., SAN JOSE, CALIFORNIA

Stress analysis report on file at GENERAL ELECTRIC CO., SAN JOSE, CALIFORNIA

DC22A6253 Rev. 0
Design specifications certified by BJORN HAABERG Prof. Eng. State Calif Reg. No. 15570

23A4920 Rev. 1
Stress analysis report certified by JOHN CARRUTH Prof. Eng. State Calif Reg. No. M21193

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 11/2 19 87 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 4/29 19 87

E R Sherrill
Inspector's Signature

Commissions N.C. 723, PA WC1766, OHIO
National Board, State, Province and No.

*Supplemental sheets in form of lists, sketches or drawings may be used provided (1) size is 8½" x 11", (2) information in items 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3, "Remarks"

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 11-10-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 23931 (3-94-047)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS. 300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	6336	N/A	S/N 6336	74	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	970	N/A	S/N 970	69	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 x 5½"	N/A	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HTH5206-3	N/A	1-8 x 5½"	N/A	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION D-11 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB H I CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 3-30, 19 94 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: R Ramey Commissions: IL932, NB7742 NIS13 (State or Province, National Board)

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 791, 965, 970, 1055, 1098, 1099, 1104
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1361

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date March 24, 19 69 Signed General Electric Co, R&FMO (Manufacturer) By [signature]

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and [illegible] of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on 3-24- 19 69, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-24- 19 69

[signature] Commissions [illegible] 206
Inspector's Signature National Board or State and No.

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 11-10-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 23932 (3-94-018)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS.300

5. (a) Construction Code ASME SEC III, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	588C	N/A	S/N 588C	67	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	A9651	N/A	S/N A9651	93	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	NONE	N/A	1-8 x 5½"	N/A	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT#52613	N/A	1-8 x 5½"	N/A	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION D-5 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&I CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 3-30, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: IL932 NB7742 NIS-B (State or Province, National Board)

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Fuel & Components Manufacturing (GE NF & CM)
2117 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : Dresden Morris, Illinois 60450
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A9651 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G005 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive , Model # 7RDB144FG001

(c) Applicable ASME Code: Section III , Edition 1974 , Addenda Date W'75 , Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 02/11/94 Signed GE - NEBG - NF & CM - QA By
(NPT Certificate Holder) (SC QA Representive)

Certificate of Authorization Expires: 6/16/96 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company , San Jose , California

Stress analysis report on file at GE Company , San Jose , California

DC22A6253 Rev. 1
Design specification certified by Bjorn Haaberg Prof. Eng. State Calif. Reg. No. 15570

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/2, 1993, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

2/11, 1994 Jerome P Emerson NC 1231, Ohio, WC 3685 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

D94-00656

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL. 60690 (Address)

Date: 11-10-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

D 23933 (3-94-049)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: D-3 CONTROL ROD DRIVE SYS.300

5. (a) Construction Code ASME SEC III, 19 65 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CONTROL ROD DRIVE	GENERAL ELECTRIC	1929	N/A	SN 1929	70	REPLACED	YES
CONTROL ROD DRIVE	GENERAL ELECTRIC	657C	N/A	SN 657C	65	REPLACEMENT	YES
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	None	N/A	1-8 X 5½"	N/A	REPLACED	NO
C.R.D. FLANGE BOLTS	GENERAL ELECTRIC	HT 52613	N/A	1-8 X 5½"	N/A	REPLACEMENT	NO

7. Description of work: REMOVE AND REPLACE CONTROL ROD DRIVE AND REPLACE CONTROL ROD DRIVE FLANGE BOLTS.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure ~~1100~~ 1040 EJK 11-10-94 psig Test Temperature ~~180~~ 200 EJK °F 11-10-94

9. Remarks: REMOVED AND REPLACED CONTROL ROD DRIVE FROM POSITION F-13 AND REPLACED 8 EA. CONTROL ROD DRIVE FLANGE BOLTS WITH NEW OR VT-1 INSPECTED BOLTS.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&I CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 3-30-95, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: IL 932, NB 7742 N I S B (State or Province, National Board)

01525

FORM N-2 MANUFACTURER'S PARTIAL DATA REPORT

Part of a Nuclear Pressure Vessel Fabricated by One Manufacturer for Another Manufacturer

As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of manufacturer of part)

(b) Manufactured for General Electric Co., APED. For use with reactor pressure vessel.
(Name and address of manufacturer of completed nuclear vessel)

2. Identification: Manufacturer's Serial No. of Part 102: 597-C, 657-C, 673-C, 724-C

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date October 25, 19 67 Signed General Electric Co. APED By G. W. Funk
(Manufacturer)

Certificate of Authorization Expires December 31, 1967

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on , 19 , and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 10-27- 1967

Inspector's Signature Commissions 17 106
National Board or State and No.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 9-7-96
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Bechtel Consturction (Name) WR 920052857 (PLAN 3-94-052)
Repair Organization P.O. No., Job No. etc.
P.P Box 829, Morris, IL 60450 (Address)

4. Identification of System: 1200 Reactor Water Clean Up

5. (a) Construction Code USAS B31.1.0/ASME Section I, 19 67/65 Edition, NO/W66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3-1201-1 Valve (8" Gate Valve)	Crane Valve	Unknown	N/A	None	N/A	REPLACED	NO
3-1201-1A Valve (2" Gate Valve)	Crane Valve	Unknown	N/A	None	N/A	REPLACED	NO
Line 3-12126-2"-A (Includes socket welded fittings and decon flange and associated bolting)	Unknown	Unknown	N/A	None	N/A	REPLACED	NO
8" Double Disc Gate Valve	Anchor Darling	ET586-1-1	N/A	SI #808C30	1993	REPLACEMENT	NO
2" Double Disc Gate Valve	Anchor Darling	ET852-3-1	N/A	SI #812C57	1993	REPLACEMENT	NO
2" Sch. 80 Pipe	Unknown	Heat E26250	N/A	SI #764C93	N/A	REPLACEMENT	NO
2" 90 Degree Socket Weld Elbows (2 Total)	Unknown	Heat LK2	N/A	SI #558B51	N/A	REPLACEMENT	NO
2" Socket Weld Tee	Unknown	Heat EC	N/A	SI #764D93	N/A	REPLACEMENT	NO

7. Description of work: Removed exisiting 8" and 2" Crane gate valves and installed 8" and 2" Anchor Darling double disc gate valves to address Gneric Letter 89-10 concerns under Moodification M12-3-92-001A. Existing 2" bypass line was also replaced under this modification.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-7 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the REPLACEMENT described in this report on 9-7, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-9-96 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

ATTACHMENT 1

NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 06

1. Owner: ComEd
One First National Plaza, Chicago IL, 60690

Date: 9-30-95

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Staion
6500 N. Dresden Road, Morris IL., 60450

Unit: 3

3. Work Performed By: BECHTEL CONSTRUCTORS (Name)

GAITHERSBURG, MARYLAND 20877 (Address)

NWR D09516 PLAN 3-94-053
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1200 REACTOR WATER CLEAN UP

5. (a) Construction Code USAS B31.1.0, 1967 Edition, NO Addenda, Code Cases NO
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NO

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
8" GATE VALVE	ANCHOR DARLING	UNKNOWN	N/A	3-1201-2	N/A	REPLACED	NO
8" GATE VALVE	ANCHOR DARLING	ET 586-2-1	N/A	3-1201-2	93	REPLACEMENT	NO

7. Description of work: Replaced existing 8" gate valve with 8" double disc gate valve per Modification M12-3-92-001B to address NRC Generic Letter 89-10 concerns.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey ISI Coordinator 9-30, 1995
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by The Hartford Steam Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 12-4, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-4-95 Inspector: A M Phinney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 06

1. Owner: ComEd
One First National Plaza, Chicago IL.,60690

Date: 2-1-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Staion
6500 N. Dresden Road, Morris IL., 60450

Unit: 3

3. Work Performed By: SAME AS ABOVE (Name)

D14546 (PLAN 3-94-055)
Repair Organization P.O. No., Job No. etc.

SAME AS ABOVE (Address)

4. Identification of System: 1500 (LPCI)

5. (a) Construction Code USAS B31.1.0, 1967 Edition, NO Addenda, Code Cases NO
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NO

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
12" 300# CHECK VALVE	C&S VALVE CO.	UNKNOWN	N/A	3-1501-63B	N/A	REPLACED	NO
12" 300# CHECK VALVE	C&S VALVE CO.	93-2216-01(Q)-01	N/A	3-1501-63B	1993	REPLACEMENT	NO
1 ⅛"-7 A-193 Grade B7 Studs	UNKNOWN	N/A	N/A	N/A	N/A	REPLACED	NO
1 ⅛"-7 A-193 Grade B7 Studs	UNKNOWN	N/A	N/A	SI #760G56	N/A	REPLACEMENT	NO
1⅛" A-194 Grade 2H Heavy Hex Nuts	UNKNOWN	N/A	N/A	N/A	N/A	REPLACED	NO
1⅛" A-194 Grade 2H Heavy Hex Nuts	UNKNOWN	HEAT 16489	N/A	SI #764D55	N/A	REPLACEMENT	NO

7. Description of work: Replace existing dual disc check valve (made by C&S Valve) with new assembly and modify flanges per Plant Change P-12-3-94-205. Longer bolting will be installed to accommodate flange stifferners.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 172 psig Test Temperature AMBIENT °F

9. Remarks: Replaced exsiting check valve per Plant Change P12-3-94-205.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 2-1 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 2-5-, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 2-5-96 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

1. Owner: Commonwealth Edison Company (Name) Date: 6-24-94
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1
2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address) Unit: 3
3. Work Performed By: BECHTEL CONSTRUCTION (Name) 3-94-056 D08408
Repair Organization P.O. No., Job No. etc.
P.O. BOX 829 MORRIS IL 60450 (Address)
4. Identification of System: 1600 (1400 CORE SPRAY)
5. (a) Construction Code ~~ASME XI CLASS 1~~ BJL 6-24-94 AISC 6TH EDITION, 19 89 Edition, 6TH Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
PENETRATION X-149B	NA	NA	NA	NA	—	REPAIR	NO
WELD BUILD ON 12" I-BEAM							

7. Description of work: WELD REINFORCEMENT OF FLUED HEAD ANCHOR STRUCTURAL STEEL OF PENETRATION X-149B (12" I BEAM & GUSSET PLATES).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure ________ psig Test Temperature ________ °F

9. Remarks: VT-3/4 EXAM PERFORMED
Flued head support had to be removed partially to accommodate installation of new bellows assembly. Support was returned to design condition. [initials] 8-3-95

Certificate of Compliance

We certify that the statements made in this report are correct and this Repair (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) August 3, 1995 (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I & I CO of HARTFORD, CT having inspected the Repair (Repair or Replacement) described in this report on 8-4, 19 ~~94~~ 95 [initials] 8-7-95 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: ~~8-4-94~~ 8-4-95 [initials] 8-7-95 Inspector: [signature] Commissions: IL 932, NB 7742 N I S B (State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-21-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: BECHTEL (Name)
P.O.BOX 829 MORRIS, IL (Address)

D08405/P12-3-92-714
Repair Organization P.O. No., Job No. etc.
PLAN # 3-94-057

4. Identification of System: 1000 SHUTDOWN COOLING

5. (a) Construction Code AISC, 19__ Edition, 6TH Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3/4" CARBON STEEL PLATE	N/A	770100	N/A	M92-04701 D92-02211	N/A	REPLACED	NO
1 1/8" HEAVY HEX NUT	N/A	797419	N/A	M92-02773 D94-00435	N/A	REPLACED	NO
5x5x1/2" TUBE STEEL	N/A	793698	N/A	M94-00644 D94-00518	N/A	REPLACED	NO
1 1/8" Ø BOLTS	N/A	800452	N/A	M94-00657 D94-00538	N/A	REPLACED	NO

7. Description of work: FLUED HEAD ANCHOR X-111A SUPPORT MEMBERS
Support was modified in order to install new bellows assembly.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure ______ psig Test Temperature ______ °F

9. Remarks: THE ABOVE MATERIAL WAS RECONCILED WITH REFERENCE TO THE CODE MATERIAL SPECIFICATION, MATERIAL RECIEPT DOCUMENTS AND THE CECO C OF C.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 7-28 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&I of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 7-24, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 7-26-95 Inspector: [illegible signature] Commissions: IL 932, NB 7792 N,I,S (State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP-11-18
REVISION 05

WR# 05348

DOCUMENT NO. 18

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7-15-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: OWNER (Name)
SAME (Address)

3-94-058 D 05348
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 / CCSW

5. (a) Construction Code B31.1.0, 1967 Edition, N/A Addenda, Code Cases N/A

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3/4-10 X 2 3/4 CAP SCREWS	UNKNOWN	NONE	NA	A-193	N/A	REPLACED	NO
~~3/4-10 X 2 3/4 CAP SCREWS~~	~~UNKNOWN~~	~~NONE~~	~~NA~~	~~A-193~~	~~NA~~	~~REPLACEMENT~~	~~NO~~
3/4-10 THREADED ROD	UNKNOWN	NONE	NA	A-193	NA	REPLACED	NO
3/4-10 THREADED ROD	UNKNOWN	NONE	NA	A-193	NA	REPLACEMENT	NO
~~3/4-10 NUTS~~	~~UNKNOWN~~	~~NONE~~	~~NA~~	~~A-194~~	~~NA~~	~~REPLACED~~	~~NO~~
3/4-10 NUTS	UNKNOWN	NONE	NA	A-194	NA	REPLACEMENT	NO

7. Description of work: REPLACE EXISTING BOLTING WITH NEW

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Replaced existing pump cover capscrews with studs and hex nuts in accordance with Minor Plant Change P12-8-93-617. Existing capscrews were replaced with longer studs in order to accommodate installing and supporting cyclone seperator.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) EST Coordinator (Title) 8-22 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+I CO of HARTFORD CT having inspected the Replacement (Repair or Replacement) described in this report on 2-2, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 2-2-95 Inspector: Robert J. Clancy Commissions: IC932, NB7742 N IS B
(State or Province, National Board)

DOCUMENT NO. 18

1. Owner: Commonwealth Edison Company (Name) WR#. 05351 Date: 7-15-94
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: CECO (Name) 3-94-061 NWR D05351
Repair Organization P.O. No., Job No. etc.
Same as above (Address)

4. Identification of System: 3-DCCSW PUMP 1500SYS

5. (a) Construction Code 831.1, 19 67 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3/4-10 X2 3/4 CAP SCREWS	UNKNOWN	NONE	N/A	A-193	N/A	REPLACED	NO
3/4-10 X2 3/4 threaded rod cap screws B7L 8-22-94	UNKNOWN	NONE	N/A	A-193	N/A	REPLACEMENT	NO
3/4-10 Hex Nut	Unknown	N/A	N/A	None	N/A	Replacement	NO

7. Description of work: REPLACE EXITING BOLTING WITH NEW

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Replaced existing pump cover capscrews with studs and hex nuts in accordance with Minor Plant Change P12-3-93-620

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 8-22- (Date), 19 94

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C Co of HARTFORD CT having inspected the Replacement (Repair or Replacement) described in this report on 2-2-, 19 95 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 2-2-95 Inspector: Robt T Kearney Commissions: IL 932, NB 7142 N I S B
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-6-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)
Gaithersberg, MD 20877 (Address)

WR 910051255 (PLAN 3-94-065)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0205 Reactor Head Spray

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2½" Gate Valve	Crane Valve	Unknown	N/A	2-0205-24	N/A	Replaced	NO
2½" Gate Valve	Anchor/Darling	ET-802-5-2	N/A	SI #813H26	1994	Replacement	NO

7. Description of work: Replaced existing gate valve with a new double disk gate valve per Plant Change P12-3-93-279 to address Generic Letter 89-10 and local leak rate leakage that exceeded administrative limits.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-6 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on 6-12, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-12-96 Inspector: [signature] Commissions: IL 937 NB 7742 N150
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-14-96
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Bechtel Constructors (Name) WR 920051827 (PLAN 3-94-066)
Repair Organization P.O. No., Job No. etc.
P.O. Box 829 Morris, IL 60450 (Address)

4. Identification of System: 0263 Reactor Vessel Water Level Instrumentation

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" X 1" 3000# SA182 Grade F304 Reducing Coupling	Unknown	Unknown	N/A	None	N/A	Replaced	No
2" X 1" 3000# SA182 Grade F304 Reducing Coupling	Alloy Stainless Products	Heat HGK	N/A	None	N/A	Replacement	No

7. Description of work: Replaced existing 2" X 1" reducing coupling under Modification M12-3-89-004A for the "A" loop of the reactor vessel water level instrumentation modification.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-17 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on 5-17, 19 96 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-24-96 Inspector: Kurt T Gainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-14-96
One First National Plaza, Chicago IL. 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Bechtel Constructors (Name) WR 920051828 (PLAN 3-94-067)
Repair Organization P.O. No., Job No. etc.
P.O. Box 829 Morris, IL 60450 (Address)

4. Identification of System: 0263 Reactor Vessel Water Level Instrumentation

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" X 1" 3000# SA182 Grade F304 Reducing Coupling	Unknown	Unknown	N/A	None	N/A	Replaced	No
2" X 1" 3000# SA182 Grade F304 Reducing Coupling	Alloy Stainless Products	Heat HGK	N/A	None	N/A	Replacement	No

7. Description of work: Replaced existing 2" X 1" reducing coupling under Modification M12-3-89-004B for the "B" loop of the reactor vessel water level instrumentation modification.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-17 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on _____, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-24-96 Inspector: Kurt T. Kaiser Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 11-10-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Owner (Name)
Same (Address)

D24111 (3-94-069)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1300 Isolation Condenser

5. (a) Construction Code ASME Section VIII, 1965 Edition, No Addenda, Code Cases None

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, No Addenda, Code Cases None

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3-1302 Isolation	Struthers Wells	66-2-5637-2	~~N/A~~ 1-4 AS	None	1967	Repair	No
Condenser Internal							
Support (Welded to			12877				
Vessel Shell)							

7. Description of work: Weld repair internal support inside of isolation condenser vessel. Support itself is not Section XI, support is welded to vessel shell.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Existing cracked weld was ground out, examined with magnetic particle, rewelded and reexamined with magnetic particle. Vessel was inspected during vessel VT-2 on 7-1-94 but does not see any pressure other than atmospheric pressure.

Certificate of Compliance

We certify that the statements made in this report are correct and this Repair (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 1-4 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C CO of HARTFORD CT having inspected the Repair (Repair or Replacement) described in this report on 3-30-95, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-30-95 Inspector: [signature] Commissions: IL932, NB7742 N/A (State or Province, National Board)

UNIT 3

4.c-c

FORM N-1 MANUFACTURERS' DATA REPORT FOR NUCLEAR VESSELS

As required by the Provisions of the ASME Code Rules

1. Manufactured by STRUTHERS WELLS CORPORATION. Warren, Pa. (Name and address of Manufacturer)

2. Manufactured for General Electric Company (Dresden) Morris, Illinois (Name and address of Purchaser)

3. Type Horiz. (Horiz. or XXX) Kind Ht. Exch (XXXXXXXXXient Ex.) Vessel No. 66-2-5637-2 (Mfrs. Serial No.) (State & State No.) Nat'l Bd. No. 12877 Yr. Built 1967

Items 4–8 incl. to be completed for xxxxxxxxxxxxxxxxxxxxxxxxxxxxxxxx shells of heat exchangers.

4. Shell: Material Fbx SA-285-C (Kind & Spec. No.) T.S. 55,000 (Min. of range specified) Nominal Thickness 5/8 in. Corrosion Allowance .100 in. Diam. 12 ft. 0 in. Length 42 ft. 5-5/8 in.

5. Seams: Long Dbl. Butt H.T.[1] No X.R. Spot Efficiency .85 % (If Class B)

Girth Dbl. Butt H.T.[1] No X.R. Spot No. of Courses 3

6. Heads (a) Material Fbx 285-C T.S. 55,000 (b) Material ______ T.S. ______

	Location (Top, bottom, ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (XXXXXor Concave)
(a)	Ends	1-3/8	132	8-3/4					Concave
(b)									

If removable, bolts used ______ (Material, Spec. No., T.S., Size, Number) Other fastening Welded Double Butt (Describe or attach sketch)

7. Jacket Closure ______ (Describe as ogee & weld, bar, etc. If bar give dimensions, describe or sketch)

8. Constructed for operating press[2] 25# Internal 1# External psi at Max. temp 300 °F Charpy Impact ______ ft-lb at temp. of ______ °F. XXXXXXX Hydrostatic or XXXXXXXXX } Test Pressure 38 psi

Items 9 and 10 to be completed for tube sections.

9. Tube Sheets: Stationary. Material F.S. 182 F1 (Kind & Spec. No.) Diam. 33 (Subject to press.) in. Thickness 4-3/4 in. Attachment Integral (Welded, Bolted)

Floating. Material ______ (Kind & Spec. No.) Diam. ______ in. Thickness ______ in. Attachment ______

10. Tubes: Material Welded Stl. SA-249-304 (Kind & Spec. No.) O.D. 1 in. Thickness .049/.083 inches or gage ______ Number 121 Type U-Tube (Straight or U)

Items 11 to 14 incl. to be completed for XXXXXXXXXXXXXXXXXXXXXXXXXXXX channels of heat exchangers.

Units 11. Shell: Material F.S. SA-182-F1 (Kind & Spec. No.) T.S. 70,000 (Min. of range specified) Nominal Thickness 4 in. Corrosion Allowance .100 in. Diam. 3 ft. 5 in. Length 2 ft. 6 in.

12. Seams: Long Sml. (Welded, Dbl., Single) H.T.[1] Yes (Yes or No) X.R. No Efficiency 100 % (If Class B)

Girth None H.T.[1] ______ X.R. ______ No. of Courses ______

13. Heads: (a) Material F.S. SA-105-2 T.S. 70,000 (b) Material ______ T.S. ______ (c) Material ______ T.S. ______

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (Convex or Concave)
(a) Top, bottom, ends	5-5/8						40-1/2	
(b) Channel								
(c) Floating								

If removable, bolts used (a) Alloy SA-193-B7-125,000-1-3/4-28 (b) ______ (c) ______ (Material, Spec. No., T.S., Size, Number) Other fastening ______ (Describe or attach sketch)

14. Constructed for specified operating press[2] 1250 psi at Max. temp 574 °F Charpy Impact ______ ft-lb at temp. of ______ °F Pneumatic Hydrostatic or Combination } Test Pressure 1615 psi

[1] If Postweld Heat-Treated

[2] List other internal or external pressures with coincident temperature when applicable.

FORM N-1 (back)

Items below to be completed for all vessels where applicable.

5. Safety Valve Outlets: Number In Line Size _____ Location _____

6. Nozzles:

	Purpose (Inlet, Outlet, Drain)	Number	Diam. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
Tube	Inlet	1	13-1/8 O.D.	W.E.	SA-182-F1	1.188	SA-182-F1	Welded
Tube	Outlet	1	9" O.D.	W.E.	SA-182-F1	1.188	SA-182-F1	Welded
Shell	Vents	2	24" O.D.	W.E.	SA-106-B	.500	SA-285-C	Welded
Shell	Fill	1	4-1/2" OD	W.E.	SA-106-B	.531	SA-285-C	Welded
Shell	Overflow	1	4-1/2 OD	W.E.	SA-106-B	.337	SA-285-C	Welded

17. Inspection Openings: Manholes, No. 1 Size 36 Location Shell
Handholes, No. _____ Size _____ Location _____
Threaded, No. 1 Size 3/4 Location Shell
S.W. 2 1" Shell

18. Supports: Skirt _____ (Yes or No) Lugs _____ (Number) Legs _____ (Number) Other Saddle (Describe) Attached Shell--welded (Where & How)

19. Remarks: Shell (Items #4 thru #8) constructed and stamped in accordance with ASME Code Section VIII

Tube Bundles and Shell hydrostatically tested separately and shipped separately.

(Brief description of service for which vessel was designed)

CERTIFICATION OF DESIGN

Design information on file at Struthers Wells Warren, Pa.

Stress analysis report on file at Struthers Wells Warren, Pa.

Design specifications certified by Warren Schultheis Prof. Eng. P.E. State Calif. Reg. No. M-11138

Stress analysis report certified by Martin L. Pomerantz Prof. Eng. P.E. State Pa. Reg. No. 9322-E

We certify that the statements made in this report are correct and that all details of material, design, construction and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date 12/15 19 67 Signed Struthers Wells Corporation (Manufacturer) By E J Honhart
E. J. Honhart

Certificate of Authorization Expires December 31st 19 67

CERTIFICATE OF SHOP INSPECTION

VESSEL MADE BY Struthers Wells Corporation at _____

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of _____ and employed by Hartford Steam Boiler Insp.& Ins. Co. of Hartford, Conn.

have inspected the pressure vessel described in this manufacturer's data report on May 8 1968, and state that to the best of my knowledge and belief, the manufacturer has constructed this pressure vessel in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____ 19 68

Inspector's Signature

Commissions N. B. 3760
National Board or State and No.

CERTIFICATE OF FIELD ASSEMBLY INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of _____ and employed by _____ of _____

have compared the statements in this manufacturer's data report with the described pressure vessel and state that parts referred to as data items _____, not included in the certificate of shop inspection have been inspected by me and that to the best of my knowledge and belief the manufacturer has constructed and assembled this pressure vessel in accordance with the ASME Code for Nuclear Vessels. The described vessel was inspected and subjected to a hydrostatic test of _____ psi.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____ 19 _____

Inspector's Signature

Commissions _____
National Board or State and No.

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5/31/94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Commonwealth Edison Co., (Name)
Dresden Nuclear Power Station (Address)

Dresden Sta., NWR D24992 (3-94-070)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: HPCI, 2300

5. (a) Construction Code USAS B31.1.0, 1967 Edition, NO Addenda, Code Cases None
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, None Addenda, Code Cases None

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	NatBrd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes /No
HPCI C/V 5, Valve Seat	Unknown	Unknown	Unknown	None	Unknown	Replaced	No
HPCI C/V 5, Valve Seat	General Electric Co.	Dwg. B704C105J	Unknown	CECO S.I. 580D79	Unknown	Replacement	No

7. Description of work: Replaced old Valve seat with new seat, seal welded new seat.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [x]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Liquid Penetrant Tested excavations, new seat and seal welds. Performed Visual Inspections per NSWP W-01. No machining was required of replacement seat.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 9-23, 1994 (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I + I CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 12-9, 1994 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-9-94 Inspector: [signature] Commissions: NB7742 N,I,S,A, IL 932 (State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 12-2-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same (Name)
Same (Address)

D19887 (3-94-071)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0220 Main Steam Drains (3-0220-2 Valve)

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" Globe Valve	Crane	N/A	N/A	3-0220-2	N/A	Replaced	NO
2" Gate Valve	Anchor Darling	ET852-1-1	N/A	3-0220-2	N/A	Replacement	NO
2" Sch.80 Pipe	Unknown	N/A	N/A	Line # 3-3007-2"-B	N/A	Replaced	NO
2" Sch.80 Pipe	Unknown	HEAT # 68610	N/A	Line # 3-3007-2"-B	N/A	Replacement	NO
2" 3000# Coupling	Unknown	HEAT # 409075	N/A	Line # 3-3007-2"-B	N/A	Replacement	NO

7. Description of work: Replaced existing globe valve with new gate valve in order to meet NRC Generic Letter 89-10 requirements. Replaced pipe with new pipe and coupling for ease of installation.

8. Test Conducted: Hydrostatic [X] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 200 °F

9. Remarks: Replaced existing valve with new valve under Minor Plant Change P12-~~94~~-~~94~~ 3-93-280. (initials) 12-2-94

Note: Valve was replaced again during D3F1B outage under 950053140 (Plan 3-95-002) (initials) 12-5-95

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 12-5 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB(CT) of HARTFORD CT having inspected the Replacement (Repair or Replacement) described in this report on 12-4-95, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-4-95 Inspector: [signature] Commissions: [illegible] (State or Province, National Board)

CATEGORY 3

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) One First National Plaza, Chicago IL. 60690 (Address)

Date: 5-30-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) 6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name) Gaithersberg, MD 20877 (Address)

WR 930053701 (PLAN 3-94-072)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 Main Steam

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
MSIV MAIN SEAT RING	CRANE	UNKNOWN	N/A	3-0203-2C	N/A	REPLACED	NO
MSIV PILOT DISC	CRANE	UNKNOWN	N/A	3-0203-2C	N/A	REPLACED	NO
MSIV MAIN DISC	CRANE	UNKNOWN	N/A	3-0203-2C	N/A	REPLACED	NO
MSIV MAIN SEAT RING	CRANE	B2890	N/A	SI #570D55	N/A	REPLACEMENT	NO
MSIV PILOT DISC	CRANE	C3561	N/A	SI #570C91	N/A	REPLACEMENT	NO
MSIV MAIN DISC	CRANE	N/A	N/A	SI #570F06	N/A	REPLACEMENT	NO

7. Description of work: Modified existing main steam isolation valve internals with upgrade kit from valve OEM due to excessive leakage during local leak rate test. Work performed in accordance with Minor Plant Change P12-3-94-214. Pilot disc replaced as routine maintenance.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure ______ psig Test Temperature ______ °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-30 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on 6/3, 19 96 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6/3/96 Inspector: [signature] Commissions: ~~IL932, NB7742NISB~~ ILL 786 NB6977 SIISB (State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 4-16-96
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: SAME AS ABOVE (Name) D23114 PLAN 3-94-073
Repair Organization P.O. No., Job No. etc.
SAME AS ABOVE (Address)

4. Identification of System: 0203 MAIN STEAM

5. (a) Construction Code USAS B312.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
MSIV MAIN SEAT RING	CRANE	UNKNOWN	N/A	VALVE 3-0203-1D	N/A	REPLACED	NO
MSIV PILOT DISC	CRANE	UNKNOWN	N/A	VALVE 3-0203-1D	N/A	REPLACED	NO
MSIV MAIN SEAT RING	CRANE	B2889	N/A	SI #570D55	N/A	REPLACEMENT	NO
MSIV PILOT DISC	CRANE	UNKNOWN	N/A	SI #570C91	N/A	REPLACEMENT	NO

7. Description of work: Modified existing main steam isolation valve internals with upgrade kit from valve OEM due to excessive leakage during local leak rate test under Minor Plant Change P12-3-94-211. Pilot disc replaced due to excessive wear (routine mainenance).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 4-16 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticu having inspected the **REPLACEMENT** described in this report on 4-17, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 4-17-96 Inspector: Kurt T. Feeney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL 60690 (Address)

Date: 3-6-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: BECHTEL CONSTRUCTORS (Name)
GAITHERSBERG, MD 20877 (Address)

D20566 PLAN 3-94-076
Repair Organization P.O. No.,J ob No. etc.

4. Identification of System: 0220 REACTOR FEEDWATER

5. (a) Construction Code USAS B31.1.0, 1967 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CRANE 18" TILTING DISC CHECK VALVE SEAT RING ASSEMBLY	CRANE VALVE	UNKNOWN	N/A	3-0220-58B	N/A	REPLACED	NO
BONNET STUD (1)	UNKNOWN	UNKNOWN	N/A	A193 Grade B7	N/A	REPLACED	NO
BONNET NUTS (2)	UNKNOWN	UNKNOWN	N/A	A194 Grade 2H	N/A	REPLACED	NO
CRANE 18" TILTING DISC CHECK VALVE SEAT RING ASSEMBLY	CRANE-ALOYCO	PT # CA00720	N/A	SI #807E71	94	REPLACEMENT	NO
BONNET STUD (1)	UNKNOWN	UNKNOWN	N/A	SI #760G26	N/A	REPLACEMENT	NO
BONNET NUTS (2)	UNKNOWN	UNKNOWN	N/A	SI #760H26	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing seat ring assembly with upgraded seat ring kit per Exempt Plant Change P12-3-93-262. Bolting was lost during valve disassembly and required replacement.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Replaced existing seat ring assembly to improve valve tightness for local leak rate testing. Bolting was lost during disassembly and was replaced.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 3-9 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 7-12, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 3-12-96 Inspector: Robert T Barney Commissions: IL932, NB7742NISB
(State or Province, National Board)

As Required by the Provisions of ASME Code Section XI

Doc #45

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL. 60690 (Address)

Date: 5-3-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: SAME AS ABOVE (Name)
SAME AS ABOVE (Address)

12896 (3-94-094)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300

5. (a) Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, Code Cases N/A

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, N/A Addenda, Code Cases N/A

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
BONNET, BOLTS	UNKNOWN	UNKNOWNE	N/A	3-2301	N/A	REPLACED	NO
BONNET, BOLTS	G.E.	813E59	N/A	3-2301	N/A	REPLACEMENT	NO

7. Description of work: Replaced control valve bolting which was galled (Eight 1½"-8 x 8" bolts).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Replaced eight steam chest studs. In-service leak test performed during Operating Surveillance DOS 2300-3 (Reactor Vessel at 920 psig).

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed Brendan J. Casey (Owner or Owner's Designee) IST Coordinator (Title) January 4 (Date) 19 95

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&CO of HARTFORD, CT. having inspected the Replacement (Repair or Replacement) described in this report on 3-30-95, 19___ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-30-95 Inspector: [signature] Commissions: IL932, NB7742 N1563
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-14-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)
" " " (Address)

NWR D20345 (3-94-097)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 HPCI

5. (a) Construction Code ASME Sect. VIII, 1965 Edition, NO Addenda, Code Cases NONE

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3-2301-23 Relief Valve	Consolidated	Unknown	N/A	None	N/A	Replaced	NO
3-2301-23 Relief Valve	Dresser Industries	TH-88699	N/A	SI* 503D99	N/A	Replacement	NO

7. Description of work: Replaced existing relief valve with a new valve assembly (Set point set/verified under NWR D23838).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure Nominal psig Test Temperature Ambient °F

9. Remarks: VT-2 performed in conjunction with DOS 2300-3.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 1-5 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C Co of HARTFORD CT having inspected the Replacement (Repair or Replacement) described in this report on 4-29, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 4-29-95 Inspector: [signature] Commissions: NB7742 NIS IL932
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL. 60690 (Address)

Date: 1-5-95

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

D25901 (3-94-098)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 HPCI

5. (a) Construction Code USAS B31.1.0, 1967 Edition, No Addenda, Code Cases None

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, No Addenda, Code Cases None

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Capscrew	Unknown	N/A	N/A	None	N/A	Replaced	NO
Hex Nut	Unknown	N/A	N/A	None	N/A	Replaced	NO
Hex Nut (7/8"-9)	Unknown	N/A	N/A	SI# 530A10	N/A	Replacement	NO
Capscrew (7/8"-9 x 5")	Unknown	N/A	N/A	SI# 790A53	N/A	Replacement	NO

7. Description of work: Replace existing bolting with longer to achieve full nut engagement.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Replaced existing bolting with longer bolting to achieve full nut engagement. Performed VT-3/4 upon completion, support is acceptable.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan A. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 1-5 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by HSBI&I Co of Hartford CT having inspected the Replacement (Repair or Replacement) described in this report on 12-5-, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-5-95 Inspector: [signature] Commissions: 11932, NB7742 NIS B (State or Province, National Board)

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL 60690 (Address)

Date: 6-1-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL, 60450 (Address)

Unit: 3

3. Work Performed By: SAME AS ABOVE (Name)

WR # 20134 (3-94-103)
Repair Organization P.O. No., Job No. etc.

SAME AS ABOVE (Address)

4. Identification of System: 2300

5. (a) Construction Code B31.1.0, 19 67 Edition, N/A Addenda, Code Cases N/A

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, N/A Addenda, Code Cases N/A

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CHECK VALVE 24"	C&S VALVE	**	N/A	3-2301-45	**	REPLACED	NO
CHECK VALVE (24")	C&S VALVE		N/A	3-2301-45	N/A	REPLCMENT	NO
	92-	2075-01-(Q)-		01			

7. Description of work: Replace existing Dual Disk check valve with brand new assembly.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: ** UNKNOWN VT-2 Examined check valve during Operating surveillance POS 2300-03 (Reactor Vessel at 920 psig). No leakage observed.

RECONC # R-94-90 CHK. VLV. RSC 6-1-94

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 7-5 (Date) 19 95

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 4-28, 19 95 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 4-28-95 Inspector: R Clancy Commissions: IL 932, NB 7742 N IS B
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-24-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 03

3. Work Performed By: Owner (Name)
Same (Address)

D26342 DOC #14 (3-94-107)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300, HPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, N/A Addenda, Code Cases N/A

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, N/A Addenda, Code Cases N/A

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
HPCI Booster Pump Suction Piping Stanchion	Not Applicable	None	N/A	M-3405-08 S.I. 501835	N/A	Repair	No

7. Description of work: 6" PIPE TO BE USED AS COUPLING AFTER CUTTING 5" STANCHION IN HALF.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure NR psig Test Temperature NR °F

9. Remarks: VT-3/4-1 Acceptable 6-24-94

Pipe stanchion was not carrying load as originally designed. Support was modified in accordance with Minor Plant Change P 12-3-94-259.

Certificate of Compliance

We certify that the statements made in this report are correct and this Repair (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 11-10 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I+I CO of HARTFORD, CT having inspected the Repair (Repair or Replacement) described in this report on 8-16-95 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 8-16-95 Inspector: Robert T. Rainey Commissions: IL 932, NB 7742 N I S B (State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7-10-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Owner (Name)
Same (Address)

D24486 (3-94-112)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500, CCSW

5. (a) Construction Code USAS B31.1.0, 1967 Edition, NA Addenda, Code Cases NA

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, ~~1989~~ NO BJC 4-23-94 Addenda, Code Cases None

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
10" CHK VLV DUO-CHECK	C&S	90-1314-010-02		3-1501-1B	1990	REPLACED	NO
10" CHK VLV. DUO-CHECK	TWR MISSION	EE69		3-1501-1B		REPLACEMENT	NO

7. Description of work: REPLACED C&S VALVE WITH TWR MISSION DUE TO SEAT LEAKAGE

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 195 psig Test Temperature 79.8 °F

9. Remarks: Replaced existing check valve due to damaged rubber seats.

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 1-24 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&I CO of HARTFORD, CT having inspected the Replacement (Repair or Replacement) described in this report on 1-26, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 1-26-96 Inspector: Kurt T Rainey Commissions: IL932, NB7742, N, I, S, B (State or Province, National Board)

ATTACHMENT 1
FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05
CATEGORY 3

1. Owner: ComEd
One First National Plaza, Chicago IL.,60690

Date: 4-1-95

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Staion
6500 N. Dresden Road, Morris IL., 60450

Unit: 3

3. Work Performed By: SAME AS ABOVE (Name)

SAME AS ABOVE (Address)

D26722 (3-94-115)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 3000 MAIN STEAM

5. (a) Construction Code ASME Section III, 19 74 Edition, S74 Addenda, Code Cases 1644, 1651, N-71, N-249
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NO

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
5/16"-24 X 1¾ CAP SCREWS	UNKNOWN	NONE	NA	NONE	NA	REPLACED	NO
5/16"-24 X 2½ CAP SCREWS	UNKNOWN	NONE	NA	SI# 814G18	NA	REPLACEMENT	NO

7. Description of work: Replace existing snubber forward bracket bolting with longer bolting to accommodate thicker spacer plate to maintain cold clearance tolerance as evaluated under Minor Plant Change P12-3-94-263.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Replaced existing cap screws on snubber Serial Number 16029 (Model PSA-3) under MPC P12-3-94-263 to maintain cold load setting within tolerance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey ISI Coordinator 4-30-96, 19__
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by The Hartford Steam Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 4-30, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 4-30-96 Inspector: Rick J. Lindsey Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7-29-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 03

3. Work Performed By: Owner (Name)

Same (Address)

D26830 (3-94-116)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300, CRD

5. (a) Construction Code ASME Section III, 19 65 Edition, W'65 Addenda, Code Cases N/A

(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, N/A Addenda, Code Cases N/A

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
CRD Flange Bolts	General Electric	N/A	N/A	1-8x5 1/2	UNK	Replaced.	~~YES~~ NO
CRD Flange Bolts	General Electric	137C9293P004		1-8x5 1/2		Replacement	NO ~~YES~~

7. Description of work: Remove and Replace CRD Flange Bolts.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: Removed eight existing capscrews which were overtorqued and replaced with brand new capscrews

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 7-23 (Date), 1994

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILLINOIS, employed by HSB I&C Co of HARTFORD, CT. have inspected the Replacement (Repair or Replacement) described in this report on 10-10-, 1994 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 10-10-94 Inspector: [signature] Commissions: IL932 NB7742 NIS-B (State or Province, National Board)

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

Doc. #16

DAP 11-18
REVISION 05

1. Owner: Commonwealth Edison Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-5-94

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
R.R. #1, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: COMMONWEALTH EDISON CO. (Name)

WR # D27264 (3-94-117)
Repair Organization P.O. No., Job No. etc.

SAME (Address)

4. Identification of System: 6600

5. (a) Construction Code ~~USAS B31.1.0~~ TEMA Class C and S&L Specification K-2183 BJC 9-5-94, 19~~67~~ BJC 9-5-94 Edition, ______ Addenda, Code Cases N/A

(b) Edition of Section XI used for Repair/Replacement 1989 Edition, NO Addenda, Code Cases N/A

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
COOLING WATER HT. EXCH	YOUNG RADIATOR	2036847	N/A	3-6669~~A~~-B BJC 1-5-95	N/A	REPLACED	NO
COOLING WATER HT. EXCH	YOUNG RADIATOR	2448587	N/A	3-6669~~A~~-B BJC 1-5-95	N/A	REPLACEMENT	NO

7. Description of work: REPLACED HEAT EXCHANGER WITH ONE FROM QUAD CITIES.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 20 psig Test Temperature Ambient °F

9. Remarks: Replaced "B" Diesel Generator Cooling Water Heat Exchanger with new assembly transferred from Quad Cities Station. Performed VT-2 exam during Operating surveillance and with cooling water pump alone. No leakage observed

Certificate of Compliance

We certify that the statements made in this report are correct and this Replacement (Repair or Replacement) Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI Coordinator (Title) 1-5 (Date), 1995

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of ILL, employed by HSB I&ICO of HARTFORD CT having inspected the Replacement (Repair or Replacement) described in this report on 12-9, 1995 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-9-95 Inspector: [signature] Commissions: IL432, NB7742 N,I,S,B (State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-26-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

WR 940097957 (PLAN 3-94-119)
Repair Organization P.O. No., Job No. etc.

Same as Above (Address)

4. Identification of System: 1300 Isolation Condeneser

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Flow Element (Annubar)	Unknown	NONE	N/A	FE-3-1341-1	N/A	REPLACED	NO
Valve	Unknown	NONE	N/A	3-1341-1-SV	N/A	REPLACED	NO
1½" 1500# Blind Flange (A182 Grade F-304)	Unknown	Heat Code C4048	N/A	SI #814B55	N/A	REPLACEMENT	NO

7. Description of work: Removed existing flow element (annubar) and associated isolation valve and installed a blind flange per Minor Plant Change P12-3-94-273.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 992 psig Test Temperature Nominal Operating °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-27 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the REPLACEMENT described in this report on 9-27, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-27-96 Inspector: Robert T. Lariviere Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 4-24-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Bechel Constructors (Name) WR 930053598 (PLAN 3-95-006)
Repair Organization P.O. No., Job No. etc.
Gaithersberg, MD 20877 (Address)

4. Identification of System: 1400 Core Spray

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
24 HEX NUTS (3/4"-10)	UNKNOWN	NONE	N/A	3-1402-4A BONNET	N/A	REPLACED	NO
VALVE BONNET	CRANE	NONE	N/A	3-1402-4A	N/A	REPLACED	NO
24 HEX NUTS (3/4"-10)	UNKNOWN	HEAT CODE BGC	N/A	SI #796D01	N/A	REPLACEMENT	NO
ANTI-CAVITATION TRIM KIT*	CONTROL COMPONENTS INC.	637721-1-KIT	N/A	SI #812A26	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing valve trim with anti-cavitation trim to address vibration problems and Generic Letter 89-10 concerns on MOV 3-1402-4A per Plant Change 12-3-93-253. *Trim kit includes new valve bonnet and bonnet studs.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 360 psig Test Temperature Ambient °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 4-24 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the REPLACEMENT described in this report on 2-24-97, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 2-24-97 Inspector: Matt Klein Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)
Gaithersberg, MD 20877 (Address)

WR 930053599 (PLAN 3-95-007)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1400 Core Spray

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
VALVE BONNET	CRANE	UNKNOWN	N/A	3-1402-4A	N/A	REPLACED	NO
VALVE BONNET STUDS	UNKNOWN	UNKNOWN	N/A	N/A	N/A	REPLACED	NO
VALVE BONNET STUD NUTS	UNKNOWN	UNKNOWN	N/A	N/A	N/A	REPLACED	NO
ANTI-CAVITATION TRIM KIT	CONTROL COMPONENTS INC.		N/A	SI #812A26*		REPLACEMENT	

7. Description of work: Replaced existing valve trim with anit-cavitation trim to address vibration problems on 3-1402-4B valve (and Generic Letter 89-10 concerns) per Plant Change P12-3-93-254. *Trim kit includes new bonnet, bonnet studs and bonnet stud nuts.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 370 psig Test Temperature 75 °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-14 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticu having inspected the **REPLACEMENT** described in this report on 5-15, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-15-97 Inspector: Robert T Lowery Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-28-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)

Same as Above (Address)

WR 950065745 (PLAN 3-95-008)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 HPCI

5. (a) Construction Code USAS B31.1.0/ASME Section III, 19 67/77 Edition, NO/S79 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" Schedule 80 A106 Grade B pipe	Unknown	Unknown	N/A	Line 3-2309-2"-L	N/A	Replaced	No
2" Schedule 80 A106 Grade B pipe	Unknown	Heat Number B45687	N/A	SI #768B31	N/A	Replacement	No

7. Description of work: Replaced existing pipe with longer pipe to ensure piping remains submerged during post-LOCA in order to provide a water seal for outboard valve (3-2301-71). This will remove the 3-2301-71 from Appendix J testing requirements. Work performed per E12-3-95-246.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 125/51.48 psig Test Temperature Ambient °F

9. Remarks: Flow verification test performed per IWC 5222(d) which is first test pressure. Weld also examined with snoop during air test (second pressure).

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-12-, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-12-97 Inspector: Kurt T Kinney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-22-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Same as Above (Name) WR 950060470 (PLAN 3-95-009)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 65 Edition, S66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6299	N/A	3-0203-4E	N/A	REPLACED	NO
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6288	N/A	SI #501G89	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing main steam safety valve (1240 set point) with rebuilt spare assembly (1260 set point).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]
Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Performed examination during system leakage test (hydro).

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-10 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-11-97, 19 ____ and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-22-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Same as Above (Name) WR 950061024 (PLAN 3-95-010)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 65 Edition, S66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6263	N/A	3-0203-4F	N/A	REPLACED	NO
6" Main Steam Safety Valve	Consolidated/Dresser	BK 7157	N/A	SI #501G89	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing main steam safety valve (1260 set point) with rebuilt spare assembly (1260 set point).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Performed examination during system leakage test (hydro).

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-13, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Kurt T. Kaiser Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-22-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Same as Above (Name) WR 950060471 (PLAN 3-95-011)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 65 Edition, S66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair. Replaced or Replacement	Code Stamped Yes/No
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6526	N/A	3-0203-4G	N/A	REPLACED	NO
6" Main Steam Safety Valve	Consolidated/Dresser	BK 7160	N/A	SI #501G89	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing main steam safety valve (1250 set point) with rebuilt spare assembly (1240 set point).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]
Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Performed examination during system leakage test (hydro).

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-13, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Robert J. Kaiser Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-22-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Same as Above (Name) WR 950061025 (PLAN 3-95-012)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 65 Edition, S66 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6532	N/A	3-0203-4H	N/A	REPLACED	NO
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6530	N/A	SI #501G89	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing main steam safety valve (1260 set point) with rebuilt spare assembly (1250 set point).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Performed examination during system leakage test (hydro).

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-10 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-10, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-10-97 Inspector: Robert T Lenz Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-11-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: ComEd/Bechtel (Name) WR 950060465 (PLAN 3-95-013)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Electromatic Relief Valve	Consolidated/Dresser	BK 7053	N/A	3-0203-3B	N/A	Replaced	No
1⅛"-12 SA194 Grade 2H Hex Nut	Unknown	Unknown	N/A	3-0203-3B	N/A	Replaced	No
Electromatic Relief Valve	Consolidated/Dresser	BK 7079	N/A	SI #809F19	N/A	Replacement	No
1⅛"-12 SA194 Grade 2H Hex Nut	Unknown	Unknown	N/A	SI #790H78	N/A	Replacement	No

7. Description of work: Replaced existing Electromatic relief valve assembly with rebuilt spare valve.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-13, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Robert T. Kinney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd/Bechtel (Name)
Same as Above (Address)

WR 950060469 (PLAN 3-95-014)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Electromatic Relief Valve	Consolidated/Dresser	BK 7082	N/A	3-0203-3E	N/A	Replaced	No
Electromatic Relief Valve	Consolidated/Dresser	BK 7050	N/A	SI #809F19	N/A	Replacement	No

7. Description of work: Replaced existing Electromatic relief valve assembly with rebuilt spare valve.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-13, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Robert T Parney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd/Bechtel (Name)
Same as Above (Address)

WR 950060661-01 (PLAN 3-95-015)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 68 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Target Rock Relief Valve	Target Rock Corp.	130A	N/A	3-0203-3A	N/A	Replaced	No
Target Rock Relief Valve	Target Rock Corp.	215	N/A	SI #570E04	N/A	Replacement	No

7. Description of work: Replaced existing Target Rock relief valve with rebuilt spare assembly.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Valve examined during system leakage test on 6/7/97, no leakage observed.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-17 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-18, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-18-97 Inspector: Robert T Reiney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-10-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

Same as Above (Address)

WR 940095273 (PLAN 3-95-016)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1400 Core Spray

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Valve Bonnet studs for 3-1402-9B Valve (1 5/8"-8 A193 Grade B7)	Unknown	Unknown	N/A	3-1402-9B	N/A	Replaced	No
Valve Bonnet stud nuts for 3-1402-9B Valve (1 5/8"-8 A194 Grade 2H)	Unknown	Unknown	N/A	3-1402-9B	N/A	Replaced	No
Valve Bonnet studs for 3-1402-9B Valve (1 5/8"-8 A193 Grade B7)	Unknown	Heat Code E9	N/A	SI #796D87	N/A	Replacement	No
Valve Bonnet stud nuts for 3-1402-9B Valve (1 5/8"-8 A194 Grade 2H)	Unknown	None	N/A	SI #760H26	N/A	Replacement	No

7. Description of work: Replaced existing bonnet bolting (studs and nuts) which were corroded with new material.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve bonnet was inspected during system leakage test on 6/7/97, no leakage was found.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-10 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-11, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: Robert T Cairney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-13-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: ComEd/Bechtel (Name) WR 950065747 (PLAN 3-95-017)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 2300 HPCI

5. (a) Construction Code USAS B31.1.0/ASME Section III, 19 67/77 Edition, NO/S77 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Line 3-2306-16"-LX (Piping Downstream of 3-2301-74 Valve)	N/A	N/A	N/A	3-2306-16"-LX	N/A	Replaced	No
16" X 12" Short Radius Reducing Elbow (A234 Grade WPB)	Consolidated Supply	Heat Code KKIB	N/A	SI #816C24	1996	Replacement	No
Two 12" 150# Pipe Flanges (A105)	Ecker-Erhardt	Serial Numbers B45473-1 and B45473-2	N/A	SI #817D57	1996	Replacement	No
12" Diameter Seamless Pipe (A106 Grade B)	Unknown	Heat A04123	N/A	SI #788B20	N/A	Replacement	No
(24) $\frac{7}{8}$"-9 Hex Nuts (A194 Grade 2H) BJC 7-15-97	Unknown	Unknown	N/A	SI #530A10	N/A	Replacement	No
(12) $\frac{7}{8}$"-9 Bolts (A193 Grade B7)	Unknown	Unknown	N/A	SI #792B77	N/A	Replacement	No

7. Description of work: Installed pair of welding neck raised flanges in HPCI turbine steam discharge piping downstream of 3-2301-74 valve to allow for local leak rate testing of valve. Special test plate (blind spectacle) installed to allow for normal operation when not testing. Work perfomed in accordance with E12-3-95-245.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure ~~N/A~~ 29 psig BJC 7-10-97 Test Temperature N/A °F

9. Remarks: New piping and bolted connection examined during HPCI surveillance DOS 2300-03 per Dresden Station Third Interval Relief Request PR-14.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 7-10 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 7-11, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 7-11-97 Inspector: Kurt T Lewis Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-19-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 3

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: ComEd\General Electric (Name) WR 950069482 (PLAN 3-95-018)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0300 Control Rod Drive

5. (a) Construction Code ASME Section III, 19 65 Edition, Winter 1965 Addenda, Code Cases 1335-2, 1361 and 1352
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive (CRD) Unit	General Electric	A2767	*	Location N-9	1979	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location N-9	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	949	*	SI #219A56	1968	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code K2VA	N/A	SI # 808E09	N/A	Replacement	No
Control Rod Drive (CRD) Unit	General Electric	286	*	Location B-9	1968	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location B-9	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	568C	*	SI #219A56	1967	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code CWQ	N/A	SI #808E09	N/A	Replacement	No

7. Description of work: Removed existing control rod drive assembly along with associated flange cap screws and replaced with refurbished spare control rod drive assembly and eight new cap screws. Existing cap screws were discarded after VT-1 examination, existing control rod drive assemblies to be refurbished and returend to Stores as spares. * See code data report.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: VT-2 examination during system leakage test.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-13 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-13, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Kurt T James Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

FORM NIS-2 SUPPLEMENT
OWNER'S REPORT OF REPAIR OR REPLACEMENT SUPPLEMENTAL SHEET

DAP 11-18
REVISION 07

1. Owner: ComEd Company (Name) Date: 5-19-97
One First National Plaza, Chicago IL, 60690 (Address)

Sheet: 2 Of 3

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: ComEd\General Electric (Name) WR 950069482 (PLAN 3-95-018)
Repair Organization P.O. No., Job No. etc.
Same as above (Address)

4. Identification of System: 0300 Control Rod Drive

5. (a) Construction Code ASME Section III, 19 65 Edition, Winter 1965 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Case NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive (CRD) Unit	General Electric	A3330	*	Location L-12	1982	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location L-12	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	979	*	SI #219A56	1969	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code CCP	N/A	SI #808E09	N/A	Replacement	No
Control Rod Drive (CRD) Unit	General Electric	30	*	Location N-4	1968	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location N-4	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	78	*	SI #219A56	1968	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code K2VA	N/A	SI #808E09	N/A	Replacement	No
Control Rod Drive (CRD) Unit	General Electric	883	*	Location J-9	1969	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location J-9	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	824	*	SI #219A56	1969	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code K2VA	N/A	SI #808E09	N/A	Replacement	No
Control Rod Drive (CRD) Unit	General Electric	A5503	*	Location L-9	1982	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location L-9	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	728C	*	SI #219A56	1967	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code CWQ	N/A	SI #808E09	N/A	Replacement	No
Control Rod Drive (CRD) Unit	General Electric	A6682	*	Location G-10	1984	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location G-10	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	944	*	SI #219A56	1969	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code CWQ	N/A	SI #808E09	N/A	Replacement	No

CATEGORY 3

FORM NIS-2 SUPPLEMENT
OWNER'S REPORT OF REPAIR OR REPLACEMENT SUPPLEMENTAL SHEET

DAP 11-18
REVISION 07

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-19-97

Sheet: 3 Of 3

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd\General Electric (Name)
Same as above (Address)

WR 950069482 (PLAN 3-95-018)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 Control Rod Drive

5. (a) Construction Code ASME Section III, 19 65 Edition, Winter 1965 Addenda, Code Cases 1335-2, 1361, 1352
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Case NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive (CRD) Unit	General Electric	605C	*	Location J-4	1967	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location J-4	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	1057	*	SI #219A56	1969	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code CWQ	N/A	SI #808E09	N/A	Replacement	No

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, Calif.
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 879, 893, 895, 897, 901, 905, 949, 1041
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1552

See sketch showing configuration and materials used. Hydro tested at 2115 psig

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date January 28, 19 69 Signed General Electric Co, APED by
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., APED, 175 Curtner Ave; San Jose, Calif.

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, Calif.

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. 9011

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13[illegible]

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this Partial Data report on 1-29 19 69, and state that to the best of my knowledge and belief, the Manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-29 19 69

[illegible] Commissions [illegible]
Inspector's Signature National Board of Boiler and Pressure Vessel Inspectors

D~~5554~~

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102; 568c 642c

(a) Constructed According to Drawing No. 237E179 G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

"See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date October 6, 19 67 Signed General Electric Co., APED by [signature]
(Manufacturer)

Certificate of Authorization Expires December 31, 1967

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. State Calif Reg. No. ME1138

Stress analysis report certified by R. L. Call Prof. Eng. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on ________ 19___, and state that to the best of my knowledge and belief the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 10-9- 19 67

[signature]
Inspector's Signature

Commissions C-1706
National Board or State and No.

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT
A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 782, 929, 934, 967, 979, 1032, 1040
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date January 31, 19 69 Signed General Electric Co, APED (Manufacturer) By [signature]

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No M11158

Stress analysis report certified by R. L. Call Prof. Eng M.E. State Calif Reg. No 13510

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on 1-31 19 69, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part inspected in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-31 19 69

[signature]
Inspector's Signature

Commissions [illegible]
National Board or State and No.

01769

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102: 8, 51, 78, 106, 107, 134

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date January 15, 1968 Signed General Electric Co, APED By W. C. Perrault
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. ___ State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. ___ State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on ___ 19___, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-16 19 68

Inspector's Signature Commissions [illegible] 706
National Board or State and No.

01318

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 295,824,956,958,962,963,969,988,997,1061,1207,1210,
NOTE: Code File 102 1215,1255,1267

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1361

See sketch showing configuration and materials used. Hydro tested at 1110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date April 2, 1969 Signed General Electric Co, R&FMO By [signature]
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on 4-4- 1969, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 4-4- 1969

[signature]
Inspector's Signature

Commissions ad706
National Board of State and No.

01509

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

As a Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel.
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102: 490-C, 728-C

(a) Constructed According to Drawing No. 237E179 G3 Drawing Prepared by H. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used, Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date October 11, 1967 Signed General Electric Co., APED by
(Manufacturer)

Certificate of Authorization Expires December 31, 1967

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design Specifications certified by W. Schultheis Prof. Eng. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of California and employed by Division of Industrial Safety Dept of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on 10/13 19 67, and state that to the best of my knowledge and belief the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 10/13 19 67

Inspector's Signature

Commissions AC 544
National Board or State and No.

0457

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 768, 843, 944, 1201,

NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1361

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date March 21, 1969 Signed General Electric Co., R&FMO, by [signature]
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on 3-21- 1969, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-21 1969

[signature] Commissions [signature]
Inspector's Signature National Board or State and No.

01874

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 888A, 900, 906, 977, 1030, 1046, 1049, 1052, 1057, 1062, 1070, 1080, 1085
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date March 5, 19 69 Signed General Electric Co, R&FMO by
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., R&FMO, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11138

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of California and employed by Division of Industrial Safety of Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19_____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 3-5- 19 69

Inspector's Signature Commissions National Board of State and Etc.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-28-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Same as Above (Name) WR 940096861 (PLAN 3-96-009)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 CCSW

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Discharge Elbow on 3B CCSW pump/Line to CCSW Vault Room Cooler	Unknown	Unknown	N/A	Line 3-1501B-10"-D/ 3-15101B-2"-D	N/A	Repair	No
$\frac{7}{8}$"-9 A193 Grade B7 Bolts	Unknown	Unknown	N/A	None	N/A	Replaced	No
$\frac{7}{8}$"-9 A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	None	N/A	Replaced	No
$\frac{7}{8}$"-9 A193 Grade B7 Bolts	Unknown	Unknown	N/A	SI #792B77	N/A	Replacement	No
$\frac{7}{8}$"-9 A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	SI #530A10	N/A	Replacement	No

7. Description of work: Repaired existing discharge elbow off of 3B CCSW pump which had some erosion. Replaced existing flange bolting with new material in order to accommodate the addition of hardened washers. Two inch socket weld on line to room cooler removed in order to gain access to elbow internal surface, reweled socket weld after repair to elbow was completed.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 225 psig Test Temperature 67 °F

9. Remarks: Examination performed during CCSW pump run.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-25 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 7-7, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 7-7-97 Inspector: Robert T. Kinney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd/Bechtel (Name)

Same as Above (Address)

WR 950063282 (PLAN 3-96-011)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0220 Reactor Feedwater

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Seat Ring Assembly for 3-0220-58A Feedwater Check Valve	Crane Valve	Unknown	N/A	3-0220-58A	N/A	Replaced	No
Bonnet Hex Nuts (1 3/8"- 8 A194 Grade 7)	Unknown	Unknown	N/A	3-0220-58A	N/A	Replaced	No
Bonnet Studs (1 3/8"- 8 A193 Grade B&)	Unknown	Unknown	N/A	3-0220-58A	N/A	Replaced	No
Seat Ring Assembly for 3-0220-58A Feedwater Check Valve	Crane Valve	C6389	N/A	SI #817B51	1996	Replacement	No
Bonnet Hex Nuts (1 3/8"- 8 A194 Grade 7)	Unknown	None	N/A	SI #796D18	N/A	Replacement	No
Bonnet Studs (1 3/8"- 8 A193 Grade B7)	Unknown	None	N/A	SI #760G26	N/A	Replacement	

7. Description of work: Replaced existing seat ring assembly on feedwater check valve with modified assembly per E12-3-95-206 to address longstanding problems with seat leakage. Hex nuts were damaged during valve disassembly and were replaced with new material. No reason given as to why studs were replaced.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-12, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-12-97 Inspector: Kurt J Carey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-11-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: ComEd/Bechtel (Name) WR 950063489 (PLAN 3-96-012)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0220 Reactor Feedwater

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Seat Ring Assembly for 3-0220-58B Feedwater Check Valve	Crane Valve	Unknown	N/A	3-0220-58B	N/A	Replaced	No
Bonnet Hex Nuts (1 3/8"- 8 A194 Grade 7)	Unknown	Unknown	N/A	3-0220-58B	N/A	Replaced	No
Seat Ring Assembly for 3-0220-58B Feedwater Check Valve	Crane Valve	C6390	N/A	SI #817B51	1996	Replacement	No
Bonnet Hex Nuts (1 3/8"- 8 A194 Grade 7)	Unknown	None	N/A	SI #796D18	N/A	Replacement	No

7. Description of work: Replaced existing seat ring assembly on feedwater check valve with modified assembly per E12-3-95-206 to address longstanding problems with seat leakage. Twelve hex nuts were damaged during valve disassembly and were replaced with new material.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-11 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-11, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: Robert T. Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-16-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd/Bechtel (Name)

Same as Above (Address)

WR 950046326 (PLAN 3-96-015)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 Main Steam

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Seat Ring Assembly Crane "Y" Pattern Globe Valve	Crane Valve	Unknown	N/A	3-0203-2B	N/A	Replaced	No
Main Disc for Crane "Y" Pattern Globe Valve	Crane Valve	Unknown	N/A	3-0203-2B	N/A	Replaced	No
Upper and lower guide liners	Crane Valve	Unknown	N/A	3-0203-2B	N/A	Replaced	No
Seat Ring Assembly Crane "Y" Pattern Globe Valve	Crane Valve	C3933	N/A	SI #570D55	1996	Replacement	No
Main Disc for Crane "Y" Pattern Globe Valve	Crane Valve	None	N/A	SI #570F06	N/A	Replacement	No
Valve Guide Liner Retrofit Kit	Crane Valve	Not Recorded	N/A	SI #812C89	N/A	Replacement	No

7. Description of work: Replaced existing seat ring assembly, main disc, and guide liner with new seat ring assembly and retrofit kit from valve OEM (Crane Valve). Existing main disc replaced with refurbished spare. Work for seat ring and guide liner replacement was per P12-3-94-213. Guide liners replaced to address longstanding problem with attachment welds cracking, seat ring was replaced due to excessive wear of stellite hardfacing.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-16 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-17, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-17-97 Inspector: Kurt T Reiner Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-16-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd/Bechtel (Name)

Same as Above (Address)

WR 950047139 (PLAN 3-96-017)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 Main Steam

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Seat Ring Assembly Crane "Y" Pattern Globe Valve	Crane Valve	Unknown	N/A	3-0203-2D	N/A	Replaced	No
Main Disc for Crane "Y" Pattern Globe Valve	Crane Valve	Unknown	N/A	3-0203-2D	N/A	Replaced	No
Upper and lower guide liners	Crane Valve	Unknown	N/A	3-0203-2D	N/A	Replaced	No
Seat Ring Assembly Crane "Y" Pattern Globe Valve	Crane Valve	Not Recorded	N/A	SI #570D55	1996	Replacement	No
Main Disc for Crane "Y" Pattern Globe Valve	Crane Valve	None	N/A	SI #570F06	N/A	Replacement	No
Valve Guide Liner Retrofit Kit	Crane Valve	Not Recorded		SI #812C89	N/A	Replacement	No

7. Description of work: Replaced existing seat ring assembly, main disc, and guide liner with new seat ring assembly and retrofit kit from valve OEM (Crane Valves). Existing main disc replaced with refurbished spare. Work for seat ring and guide liner replacement was per P12-3-94-215. Guide liners replaced to address longstanding problem with attachment welds cracking, seat ring was replaced due to excessive wear of stellite hardfacing.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-16 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-17, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-17-97 Inspector: Kurt T. Bürney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 10-31-96
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 2/3

3. Work Performed By: Same as Above (Name) WR 950097245 (PLAN 3-96-019)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Inlet Flange Studs (1⅝" X 7¼") SA193 Grade B7	Unknown	N/A	N/A	None	N/A	Replaced	NO
Inlet Flange Studs (1⅝" X 7¼") SA193 Grade B7	Dresser	Heat Numbers QT60 and QT61	N/A	SI #570C07	N/A	Replacement	NO

7. Description of work: Replaced lost inlet flange studs.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 10-31 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on 11-11, 19 96 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 11-11-96 Inspector: Robert T. Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 9-24-96
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Same as Above (Name) WR 960049833 (PLAN 3-96-020)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 CCSW

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3A CCSW Pump Casing Bolts (¾"-10 x 4")	Unknown	N/A	N/A	3A-1501-44	N/A	REPLACED	NO
3A CCSW Pump Casing Bolts (¾"-10 x 4")	Unknown	HEAT 8090125	N/A	SI #530A04	N/A	REPLACEMENT	NO

7. Description of work: Replaced some of the existing pump casing bolts in order to accommodate hardened washers. Existing bolts would not achieve full nut engagment so longer bolting was installed.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: Pump casing was examined at nominal operating pressure for leaks.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey ISI COORDINATOR 9-24, 19 96
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford; Connecticu having inspected the **REPLACEMENT** described in this report on 9-25, 19 96 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-25-96 Inspector: Robert T Reiney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-28-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: ComEd\Bechtel (Name) WR 930053801 (PLAN 3-96-021)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
A194 Grade 2H Heavy Hex nuts ($1\frac{1}{8}$"-7)	Unknown	Unknown	N/A	3-1501-63C	N/A	Replaced	No
A193 Grade B7 Studs ($1\frac{1}{8}$"-7)	Unknown	Unknown	N/A	3-1501-63C	N/A	Replaced	No
A194 Grade 2H Heavy Hex nuts ($1\frac{1}{8}$"-7)	Unknown	Heat HFF		SI #764D55	N/A	Replacement	No
A193 Grade B7 Studs ($1\frac{1}{8}$"-7)	Unknown	Unknown		SI #760G56	N/A	Replacement	No
A194 Grade 7 Coupling Rod	Unknown	Unknown		SI #819F03	N/A	Replacement	No

7. Description of work: Replaced existing 3C LPCI pump discharge check valve bolting under E12-3-95-234. Piping was configured such that some of the boltin could not be removed. Modification allows installation of stud couplings.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 160 psig Test Temperature 68 °F

9. Remarks: Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-10 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-11, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: Kurt T. Ramm Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-28-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: ComEd\Bechtel (Name) WR 930053800 (PLAN 3-96-022)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
A194 Grade 2H Heavy Hex nuts ($1\frac{1}{8}$"-7)	Unknown	Unknown	N/A	3-1501-63D	N/A	Replaced	No
A193 Grade B7 Studs ($1\frac{1}{8}$"-7)	Unknown	Unknown	N/A	3-1501-63D	N/A	Replaced	No
A194 Grade 2H Heavy Hex nuts ($1\frac{1}{8}$"-7)	Unknown	Heat HFF		SI #764D55	N/A	Replacement	No
A193 Grade B7 Studs ($1\frac{1}{8}$"-7)	Unknown	Unknown		SI #760G56	N/A	Replacement	No
A194 Grade 7 Coupling Rod	Unknown	Unknown		SI #819F03	N/A	Replacement	No

7. Description of work: Replaced existing 3D LPCI pump discharge check valve bolting under E12-3-95-235. Piping was configured such that some of the boltin could not be removed. Modification allows installation of stud couplings.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 165 psig Test Temperature 68 °F

9. Remarks: Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-4 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-11, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: Robert T. Reimers Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-23-96

Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechel Constructors (Name)

WR 960030678 (PLAN 3-96-023)
Repair Organization P.O. No., Job No. etc.

Gaithersberg, MD 20877 (Address)

4. Identification of System: 1500 CCSW/LPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
SUPPORT M-1200D-311 (NEW SUPPORT, SEE BELOW FOR MATERIALS)	BECHTEL	NONE	N/A	M-1200D-311	1996	REPLACEMENT	NO
HEAT EXCHANGER CLAMP	LISEGA	NONE	N/A	SI #817F35	N/A	REPLACEMENT	NO
STRUT ASSEMBLY	LISEGA	NONE	N/A	SI #817E35	N/A	REPLACEMENT	NO
END BRACKET	LISEGA	A 8801	N/A	SI #817D35	N/A	REPLACEMENT	NO
1" X 8" X 6" A36 PLATE	UNKNOWN	HT 351799	N/A	SI #784G82	N/A	REPLACEMENT	NO

7. Description of work: Added two new Class 3 supports as part of Unit 3 East LPCI corner room steel modification (E12-3-95-258).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-23 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 9-25, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-25-96 Inspector: Kurt T. Kairys Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-16-96

Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechel Constructors (Name)

Gaithersberg, MD 20877 (Address)

80 BAK 9-23-96
WR 96003067~~8~~ (PLAN 3-96-024)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 CCSW/LPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
SUPPORT M-1200D-309 (NEW SUPPORT, SEE BELOW FOR MATERIALS)	BECHTEL	NONE	N/A	M-1200D-309	1996	REPLACEMENT	NO
HEAT EXCHANGER CLAMP	LISEGA	NONE	N/A	SI #817F35	N/A	REPLACEMENT	NO
STRUT ASSEMBLY	LISEGA	NONE	N/A	SI #817H34	N/A	REPLACEMENT	NO
END BRACKET	LISEGA	NONE	N/A	SI #817D35	N/A	REPLACEMENT	NO
1" X 18" X 18" A36 PLATE	UNKNOWN	NONE	N/A	SI #770F00	N/A	REPLACEMENT	NO

7. Description of work: Added two new Class 3 supports as part of Unit 3 West LPCI corner room steel modification (E12-3-95-259).

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-16 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticu having inspected the **REPLACEMENT** described in this report on 9-17, 19 96 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-17-96 Inspector: Kurt T Carroll Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd/Bechtel (Name)
Same as Above (Address)

WR 960057668 (PLAN 3-96-026)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 HPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Check Valve bonnet studs (5/8"-11 X 7" A194 Grade B7)	Unknown	Unknown	N/A	Valve 3-2301-50A	N/A	Replaced	No
Check Valve bonnet stud hex nuts (5/8"-11 A193 Grade 2H)	Unknown	Unknown	N/A	Valve 3-2301-50A	N/A	Replaced	No
Check Valve bonnet studs (5/8"-11 X 7" A194 Grade B7)	Unknown	Unknown	N/A	SI #796C99	N/A	Replacement	No
Check Valve bonnet stud hex nuts (5/8"-11 A193 Grade 2H)	Unknown	Unknown	N/A	SI #500E52	N/A	Replacement	No

7. Description of work: Replaced existing bonnet bolting to accommodate installation of hardened washers.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve examined during system operation for leakage, no leakage detected.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-18 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-19, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-19-97 Inspector: Robert T. Parsons Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-23-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)
Gaithersberg, MD 20877 (Address)

WR 960030678 (PLAN 3-96-028)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 CCSW/LPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
SUPPORT M-3413-13	UNKNOWN	NONE	N/A	M-3413-13	N/A	REPLACEMENT	NO
SUPPORT M-3413-13	UNKNOWN	NONE	N/A	M-3413-13	N/A	REPLACEMENT	NO

7. Description of work: In order to raise 3A LPCI heat exchanger as part of East LPCI corner room steel modification, support had to be temporarily removed. Fillet weld at tube steel-to-plate was ground out in order to slide box guide out of the way and raise heat exchanger. Once heat exchanger was in place, support was rewelded using existing materials.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-23 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 9-25, 19 96 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-25-96 Inspector: Scott T Reiner Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-19-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd\General Electric (Name)
Same as Above (Address)

WR 960049569 (PLAN 3-96-030)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 Control Rod Drive

5. (a) Construction Code ASME Section III, 19 65 Edition, Winter 1965 Addenda, Code Cases 1335-2, 1361 and 1352
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive (CRD) Unit	General Electric	A3180	*	Location H-7	1980	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location H-7	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	498C	*	SI #219A56	1967	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code K2VB	N/A	SI # 808E09	N/A	Replacement	No

7. Description of work: Removed existing control rod drive assembly along with associated flange cap screws and replaced with refurbished spare control rod drive assembly and eight new cap screws. Existing cap screws were discarded after VT-1 examination, existing control rod drive assemblies to be refurbished and returend to Stores as spares. * See code data report.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: VT-2 examination during system leakage test.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-13 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-13, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Kurt T. Kairn Commissions: IL932, NB7742NISB
(State or Province, National Board)

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 102; 317-C, 498-C, 709-C

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date October 30, 19 67 Signed General Electric Co. APED (by) [signature]
(Manufacturer)

Certificate of Authorization Expires December 31, 1967

CERTIFICATION OF DESIGN

Design information on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications verified by W. Schultheis Prof. Eng. State Calif Reg. No. M1153

Stress analysis report certified by R. L. Call Prof. Eng. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the state of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19 _____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the equipment described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 11-1- 19 67

[signature]
Inspector's Signature

Commissions _____
National Board or State and No.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-19-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd\General Electric (Name)
Same as Above (Address)

WR 960049569 (PLAN 3-96-031)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0300 Control Rod Drive

5. (a) Construction Code ASME Section III, 19 65 Edition, Winter 1965 Addenda, Code Cases 1335-2, 1361 and 1352
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Control Rod Drive (CRD) Unit	General Electric	537C	*	Location G-7	1967	Replaced	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Unknown	N/A	Location H-7	N/A	Replaced	No
Control Rod Drive (CRD) Unit	General Electric	806	*	SI #219A56	1967	Replacement	Yes
CRD Flange Cap Screws (8 Total)	General Electric	Heat Code K2VB	N/A	SI # 808E09	N/A	Replacement	No

7. Description of work: Removed existing control rod drive assembly along with associated flange cap screws and replaced with refurbished spare control rod drive assembly and eight new cap screws. Existing cap screws were discarded after VT-1 examination, existing control rod drive assemblies to be refurbished and returend to Stores as spares. * See code data report.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: VT-2 examination during system leakage test.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-13 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-13, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Kurt T. Kalmes Commissions: IL932, NB7742NISB
(State or Province, National Board)

01724

FORM N-2 MANUFACTURERS' PARTIAL DATA REPORT

A Part of a Nuclear Vessel Fabricated by One Manufacturer for Another Manufacturer
As required by the Provisions of the ASME Code Rules

1. (a) Manufactured by General Electric Co., APED, 175 Curtner Ave; San Jose, California
(Name and address of Manufacturer of part)

(b) Manufactured for General Electric Co. For use with reactor pressure vessel
(Name and address of Manufacturer of completed nuclear vessel)

2. Identification-Manufacturer's Serial No. of Part 806, 876, 878, 887, 930, 946, 959, 995
NOTE: Code File 102

(a) Constructed According to Drawing No. 237E179-G3 Drawing Prepared by D. L. Peterson

(b) Description of Part Inspected Control Rod Drive

3. Remarks: Fabricated and inspected in accordance with Section III and applicable code cases no. 1335-2, 1361, and 1352

See sketch showing configuration and materials used. Hydro tested at 2110 psi

(Brief description of service for which vessel part was designed)

We certify that the statements made in this report are correct and that all details of material, design, construction, and workmanship of this pressure vessel conform to the ASME Code for Nuclear Vessels.

Date January 24, 19 69 Signed General Electric Co. APED By
(Manufacturer)

Certificate of Authorization Expires December 31, 1970

CERTIFICATION OF DESIGN

Design information of file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Stress analysis report on file at General Electric Co., APED, 175 Curtner Ave; San Jose, California

Design specifications certified by W. Schultheis Prof. Eng. M.E. State Calif Reg. No. M11158

Stress analysis report certified by R. L. Call Prof. Eng. M.E. State Calif Reg. No. 13540

CERTIFICATE OF SHOP INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of California and employed by Division of Industrial Safety Dept. of Industrial Relations have inspected the part of a pressure vessel described in this manufacturer's partial data report on _____ 19_____, and state that to the best of my knowledge and belief, the manufacturer has constructed this part in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in this manufacturer's partial data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 1-24- 19 69

M. A. Dawes Commissions C-1700
Inspector's Signature National Board or State and No.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-24-96

Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)
Gaithersberg, MD 20877 (Address)

WR 950063471 (PLAN 3-96-032)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 High Pressure Coolant Injection

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
5/8" A193 Grade B7 Studs	Unknown	N/A	N/A	None	N/A	REPLACED	NO
5/8" A194 Grade 2H Hex Nuts	Unknown	N/A	N/A	None	N/A	REPLACED	NO
1/4" A193 Grade B7 Studs	Unknown	N/A	N/A	None	N/A	REPLACED	NO
1/4" A194 Grade 2H Hex Nuts	Unknown	N/A	N/A	None	N/A	REPLACED	NO
4" X 6" 150# Relief Valve	Dresser	N/A	N/A	3-2301-53	N/A	REPLACED	NO

7. Description of work: Replaced existing relief valve with a tested spare and replaced inlet and discharge flange bolting with new material to accommodate hardened washers.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 43 psig Test Temperature Ambient °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-24 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 9-25, 19 96 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-25-96 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

FORM NIS-2 SUPPLEMENT
OWNER'S REPORT OF REPAIR OR REPLACEMENT SUPPLEMENTAL SHEET

DAP 11-18
REVISION 07

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-24-96

Sheet: 2 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Construction (Name)
Gaithersberg, MD 20877 (Address)

WR 950063471 (PLAN 3-96-032)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 High Pressure Coolant Injection

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Case NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
5/8" A193 Grade B7 Studs	Unknown	N/A	N/A	SI #500E52	N/A	REPLACEMENT	NO
5/8" A194 Grade 2H Hex Nuts	Unknown	Heat CJW	N/A	SI #796C99	N/A	REPLACEMENT	NO
3/4" A193 Grade B7 Studs	Unknown	Heat BTM and CJX	N/A	SI #796D01	N/A	REPLACEMENT	NO
3/4" A194 Grade 2H Hex Nuts	Unknown	Heat BMP	N/A	SI #796D75	N/A	REPLACEMENT	NO
4" X 6" 150# Relief Valve	Dresser	TH87924	N/A	SI #790E62	N/A	REPLACEMENT	NO

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-6-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2

3. Work Performed By: Bechtel Constuctors (Name)

Gaithersberg, MD 20877 (Address)

WR 960030678 (PLAN 3-96-033)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 Low Pressure Coolant Injection

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N-416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3A LPCI Heat Exchanger shell side drain line socket weld (2" NPS Sch. 80 Carbon Steel)	Unknown	Unknown	N/A	None	N/A	REPLACED	NO

7. Description of work: Removed exisiting shell side drain piping at socket weld in order to install supports on 3A LPCI heat exchanger as part of East LPCI Corner Room Structural Steel modification (E12-3-95-258). Upon completion of support installation, original shell side drain line was reinstalled by socket welding.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 125 psig Test Temperature Ambient °F

9. Remarks: In accordance with Dresden Station Third Interval Relief Request PR-14.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-23 (Date), 1996

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on 9-25, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-25-96 Inspector: Kurt T. Kenny Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 9-24-96

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

Same as Above (Address)

WR 960081175 (PLAN 3-96-034)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 High Pressure Coolant Injection

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3A CONTAINMENT COOLING HEAT EXCHANGER	BERLIN CHAPMAN	3007	3007	3A-1503	1967	REPAIR	YES

7. Description of work: Restored existing baffle plate-to-vessel shell weld that was reduced while Mechanical Maintenance was grinding out non-Code weld inside of heat exchanger. Weld repair did not extend into vessel wall over 10% of vessel nominal thickness.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: Weld repair did not extend into the heat exchanger shell (< 10% of vessel wall) and is exempted from pressure testing per IWA-4700).

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 9-24 (Date), 19 96

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPAIR** described in this report on 9-24, 1996 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 9-25-96 Inspector: Robert T Lewis Commissions: IL932, NB7742NISB
(State or Province, National Board)

UNIT 3 EAST "A" LOOP

FORM U-1 MANUFACTURERS' DATA REPORT FOR UNFIRED PRESSURE VESSELS

As required by the Provisions of the ASME Code Rules

1. Manufactured by Berlin Chapman, Div. Perfex Corp., Berlin, Wisconsin (Name and address of Manufacturer)

2. Manufactured for General Electric Company, San Jose, California (Name and address of Purchaser)

3. Type Vertical (Horiz. or Vert.) Kind Heat Exch. (Tank, Jacketed, Heat Exch.) Vessel No. 05036-4 (Mfrs. Serial) (State & State No.) Natl. Bd. No. 3007 Yr. Built 1967

Items 4-9 incl. to be completed for single wall vessels, jackets of jacketed vessels, or shells of heat exchangers.

4. SHELL: Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Fig. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 21 Ft. 4 5/8 In.

5. SEAMS: Long Dbl.Weld butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl.Weld butt H.T. No X.R. Complete Sectioned No No. of Courses 3

If riveted describe seams fully on reverse side of form.

6. HEADS (a) Material See Item 10 T.S. (b) Material T.S.

Location (Top, bottom, ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a)								
(b)								

If removable, bolts used (Material, Spec. No., T.S., Size, Number) Other fastening (Describe or Attach Sketch)

7. STAYBOLTS: (Material) If hollow (Size of Hole) Attachment (Threaded, Welded) Pitch (Horiz.) X (Vert.) Diam. (Nominal)

8. JACKET CLOSURE: (Describe as ogee & weld, bar, etc. If bar, give dimensions, if bolted, describe or sketch)

9. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than -20°) °F. Hydrostatic ~~Pneumatic or Combination~~ Test Press 563 psi.

Items 10 and 11 to be completed for tube sections.

10. TUBE SHEETS: (2) Stationary. Material SA-212-B FBX (Kind & Spec. No.) Diam. 62 3/4 In. (Subject to Pressure) Thickness 3 3/4 In. Attachment Welded (Welded, Bolted)

Floating. Material (Kind & Spec. No.) Diam. In. Thickness In. Attachment

11. TUBES: Material 70-30 CU.NI. SB-111 (Kind & Spec. No.) O.D. 3/4 In. Thickness 18 BWG Inches or Gage Number 2512 Type Straight (Straight or U)

Items 12-15 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers. (2)

12. SHELL Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Fig. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 2 - 3 1/2 1 Ft. 11 3/4

13. SEAMS: Long Dbl.Weld Butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl.Weld Butt H.T. No X.R. Complete Sectioned No No. of courses 1

If riveted describe seams fully on reverse side of form.

14. HEADS (a) Material T.S. (b) Material SA-212-B T.S. 70000 (c) Material SA-212-B T.S. 70000

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a) Top, bottom, ends								
(b) Channel Top	8"						70 1/8"	
(c) Floating Bot.	5 3/4"						70 1/8"	

If removable, bolts used (a) (Material, Spec. No., T.S., Size, Number) (b) 60 - 1 3/8" SA-193 B7 125000

(c) 60 - 1 3/8" SA-193 B7 125000 Other fastening (Describe or Attach Sketch)

15. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than -20°) °F. Hydrostatic ~~Pneumatic or Combination~~ Test Press 563 psi.

Items below to be completed for all vessels where applicable.

16. SAFETY VALVE OUTLETS: Number Size Location

17. NOZZLES

Purpose (Inlet, Outlet, Drain)	Number	Diam. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
Inlet-Out	2	24"x18" Con.Red	Weld End	SA-106-B	.500"	SA-212-B	Welded
Inlet-Out	2	16"	Weld End	SA-106-B	.500"	SA-212-B	Welded
Vents	3	1"	Socket Weld	SA-181-1	6000#	Coupling	Welded
Drains	2	2"	Socket Weld	SA-181-1	6000#	Coupling	Welded

* If postweld heat treated ... List under remarks other internal or external pressures with coincident temperatures when applicable.

FORM U-1 (back)

18. INSPECTION OPENINGS: Manhole, No. _____ Size _____ Location _____
Handhole, No. _____ Size _____ Location Removable Heads
Threaded, No. _____ Size _____ Location _____

19. SUPPORTS: Skirt No (Yes or No) Legs _____ (Number) Legs _____ (Number) Other Brackets (Describe) Attached Weld to (Welded or How) shel

20. REMARKS: Containment Exchanger Item 1
Shell Side - Demineralized Water - Tube Side - River Water
Channel Side of Tube Sheets & Monel Overlay.
Channel Flgs. SA-105-2 5 5/16" tk.

(Brief description of purpose of the vessel, as Air Tank, After Cooler, Jacketed Cooker, etc. State contents of each part.)

We certify that the statements made in this report are correct and that all details of design, material, construction, and workmanship of this vessel conform to the ASME Code for Unfired Pressure Vessels.

Date AUG 3 1967 19 ____ Signed Berlin Chapman (Manufacturer) By [signature]

Certificate of Authorization Expires December 31, 1970

CERTIFICATE OF SHOP INSPECTION

VESSEL MADE BY Berlin, Chapman at Berlin, Wisconsin

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of N.B. and employed by Hartford Steam Boiler I. & D. Co. of Hartford, Conn. have inspected the pressure vessel described in this manufacturer's data report on AUG 31 1967 19 ___, and state that to the best of my knowledge and belief, the manufacturer has constructed this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date AUG 3 19 ___

[signature] Commissions N.B. 1492
Inspector's Signature Nat'l Board or State and No.

CERTIFICATE OF FIELD ASSEMBLY INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of _____ and employed by _____ of _____ have compared the statements in this manufacturer's data report with the described pressure vessel and state that parts referred to as data items _____ not included in the certificate of shop inspection have been inspected by me and that to the best of my knowledge and belief the manufacturer has constructed and assembled this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code. The described vessel was inspected and subjected to a hydrostatic test of _____ psi.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____ 19 ___

_____ Commissions _____
Inspector's Signature Nat'l Board or State and No.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-28-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: ComEd/Bechtel (Name) WR 950063469 (PLAN 3-96-038)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI

5. (a) Construction Code ASME Section VIII, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" Relief Valve	Dresser Industries	UNKNOWN	N/A	3-1501-17A	N/A	Replaced	No
2" Relief Valve	Dresser Industries	TL-05316	N/A	SI #503E01	N/A	Replacement	No

7. Description of work: Replaced existing LPCI "A" discharge relief valve which failed IST surveillance with brand new valve.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]
Test Pressure 160 psig Test Temperature 77 °F

9. Remarks: Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-28 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 5-30, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-30-97 Inspector: Kurt T Casey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 2-13-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: Spare
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Same as Above (Name) WR 960011774 (PLAN 3-97-001)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 68 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Valve Disc for 6" Main Steam Safety Valve	Consolidated/Dresser	Unknown	N/A	Part #VGZ815K-OS343	N/A	Replaced	No
Valve Disc for 6" Main Steam Safety Valve	Consolidated/Dresser	Heat G1624	N/A	SI #570D97	N/A	Replacement	No

7. Description of work: Replaced disc for main steam safety relief valve (Serial Number BK6264) which was obtained from Oyster Creek and rebuilt by ComEd Dresden Mechanical Maintenance Department. Valve to be shipped to Quad Cities Station as a spare assembly.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure _____ psig Test Temperature _____ °F

9. Remarks: Valve set point and seat leakage tests performed at Dresden per DMP 0200-30.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey ISI COORDINATOR 2-13, 1997
(Owner or Owner's Designee) (Title) (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 2-20, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 2-20-97 Inspector: _____ Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-20-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: ComEd\Bechtel (Name) WR 960118198 (PLAN 3-97-002)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI/CCSW

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3A CCSW/LPCI Heat Exchanger	Berlin-Chapman	05036-4	3007	3A-1503	1967	Repair	Yes
Upper channel partition plate (A285 Grade C)	Unknown	None	N/A	Carbon Steel	N/A	Replaced	No
Upper channel partition plate (A240 Type 316L)	Unknown	Heat Number 64892	N/A	SI #819B22	N/A	Replacement	No

7. Description of work: Replaced existing carbon steel partition plate in upper channel with stainless steel plate in accordance with E12-3-96-228 and repaired pitted areas in shell wall by weld build-up.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 108/160 psig Test Temperature 59/61 °F

9. Remarks: Test pressure and test temperature are tube side and shell side respectively. Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-27 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPAIR/REPLACEMENT** described in this report on 5-27, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-27-97 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

UNIT 3 EAST "A" LOOP

FORM U-1 MANUFACTURERS' DATA REPORT FOR UNFIRED PRESSURE VESSELS

As required by the Provisions of the ASME Code Rules

1. Manufactured by Berlin Chapman, Div. Perfex Corp., Berlin, Wisconsin (Name and address of Manufacturer)

2. Manufactured for General Electric Company, San Jose, California (Name and address of Purchaser)

3. Type Vertical (Horiz. or Vert.) Kind Heat Exch. (Tank, Jacketed, Heat Exch.) Vessel No. 05036-4 (Mfrs. Serial) ( ) (State & State No.) Natl. Bd. No. 3007 Yr. Built 1967

Items 4–9 incl. to be completed ~~for single wall vessels (such as air tanks), jackets of jacketed vessels~~ or shells of heat exchangers.

4. SHELL: Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Fig. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 21 Ft. 4 5/8 In.

5. SEAMS: Long Dbl. Weld butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl. Weld butt H.T. No X.R. Complete Sectioned No No. of Courses 3

If riveted describe seams fully on reverse side of form.

6. HEADS (a) Material See Item 10 T.S. (b) Material T.S.

Location (Top, bottom, ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a)								
(b)								

If removable, bolts used (Material, Spec. No., T.S., Size, Number) Other fastening (Describe or Attach Sketch)

7. STAYBOLTS: (Material) If hollow (Size of Hole) Attachment (Threaded, Welded) Pitch (Horiz.) X (Vert.) Diam. (Nominal)

8. JACKET CLOSURE: (Describe as ogee & weld, bar, etc. If bar, give dimensions, if bolted, describe or sketch)

9. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than −20°) °F. Hydrostatic ~~Pneumatic or Combination~~ Test Press. 563 psi.

Items 10 and 11 to be completed for tube sections.

10. TUBE SHEETS: (2) Stationary. Material SA-212-B FBX (Kind & Spec. No.) Diam. 62 3/4 (Subject to Pressure) In. Thickness 3 3/4 In. Attachment Welded (Welded, Bolted)

Floating. Material (Kind & Spec. No.) Diam. In. Thickness In. Attachment

11. TUBES: Material 70-30 CU.NI. SB-111 (Kind & Spec. No.) O.D. 3/4 In. Thickness 18 BWG Inches or Gage Number 2512 Type Straight (Straight or U)

Items 12–15 incl. to be completed ~~for inner chambers of jacketed vessels or~~ channels of heat exchangers. (2)

12. SHELL Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Fig. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 2 – 3 1/2 / 1 Ft. 11 3/4

13. SEAMS: Long Dbl. Weld Butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl. Weld Butt H.T. No X.R. Complete Sectioned No No. of courses 1

If riveted describe seams fully on reverse side of form.

14. HEADS (a) Material T.S. (b) Material SA-212-B T.S. 70000 (c) Material SA-212-B T.S. 70000

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a) Top, bottom, ends								
(b) Channel Top	8"						70 1/8"	
(c) ~~Floating~~ Bot.	5 3/4"						70 1/8"	

If removable, bolts used (a) (Material, Spec. No., T.S., Size, Number) (b) 60 – 1 3/8" SA-193 B7 125000

(c) 60 – 1 3/8" SA-193 B7 125000 Other fastening (Describe or Attach Sketch)

15. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than −20°) °F. Hydrostatic ~~Pneumatic or Combination~~ Test Press. 563 psi.

Items below to be completed for all vessels where applicable.

16. SAFETY VALVE OUTLETS: Number Size Location

17. NOZZLES

Purpose (Inlet, Outlet, Drain)	Number	Diam. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
Inlet-Out	2	24"x18" Con. Red	Weld End	SA-106-B	.500"	SA-212-B	Welded
Inlet-Out	2	16"	Weld End	SA-106-B	.500"	SA-212-B	Welded
Vents	3	1"	Socket Weld	SA-181-1	6000#	Coupling	Welded
Drains	2	2"	Socket Weld	SA-181-1	6000#	Coupling	Welded

* If postweld heat-treated. ** List under remarks other internal or external pressures with coincident temperature when applicable.

FORM U-1 (back)

18. INSPECTION OPENINGS: Manholes, No. _____ Size _____ Location _____
Handholes, No. _____ Size _____ Location Removable Heads
Threaded, No. _____ Size _____ Location _____

19. SUPPORTS: Skirt No (Yes or No) Legs _____ (Number) Legs _____ (Number) Other Brackets (Describe) Attached Weld to (Welded or Bolted) shel

20. REMARKS: Containment Exchanger Item 1
Shell Side - Demineralized Water - Tube Side - River Water
Channel Side of Tube Sheets & Monel Overlay.

Channel Plgs. SA-105-2 5 5/16" tk.

(Brief description of purpose of the vessel, as Air Tank, After Cooler, Jacketed Cooker, etc. Note contents of each part.)

We certify that the statements made in this report are correct and that all details of design, material, construction, and workmanship of this vessel conform to the ASME Code for Unfired Pressure Vessels.

Date AUG 3 1967 19 Signed Berlin Chapman (Manufacturer) By [signature]

Certificate of Authorization Expires December 31, 1970

CERTIFICATE OF SHOP INSPECTION

VESSEL MADE BY Berlin, Chapman at Berlin, Wisconsin

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of U.S. and employed by Hartford Steam Boiler I. & D. Co. of Hartford, Conn. have inspected the pressure vessel described in this manufacturer's data report on AUG 31 1967 19 , and state that to the best of my knowledge and belief, the manufacturer has constructed this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date AUG 3 1967 19

[signature] Commissions N.B. 1492
Inspector's Signature Nat'l Board or State and No.

CERTIFICATE OF FIELD ASSEMBLY INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of _____ and employed by _____ of _____ have compared the statements in this manufacturer's data report with the described pressure vessel and state that parts referred to as data items _____ not included in the certificate of shop inspection have been inspected by me and that to the best of my knowledge and belief the manufacturer has constructed and assembled this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code. The described vessel was inspected and subjected to a hydrostatic test of _____ psi.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date _____ 19 _____

_____ Commissions _____
Inspector's Signature Nat'l Board or State and No.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-21-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd\Bechtel (Name)

Same as Above (Address)

WR 960118148 (PLAN 3-97-003)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 LPCI/CCSW

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3B CCSW/LPCI Heat Exchanger	Berlin-Chapman	05036-3	3006	3B-1503	1967	Repair	Yes
Upper channel partition plate (A285 Grade C)	Unknown	None	N/A	Carbon Steel	N/A	Replaced	No
Upper channel partition plate (A240 Type 316L)	Unknown	Heat Number not recorded	N/A	SI #819B22	N/A	Replacement	No

7. Description of work: Replaced existing carbon steel partition plate in upper channel with stainless steel plate in accordance with E12-3-96-229 and repaired pitted areas in shell wall by weld build-up.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 165/175 psig Test Temperature 68/58 °F

9. Remarks: Test pressure and temperature listed for shell side and tube side respectively. Inspection performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-28 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticu having inspected the REPAIR/REPLACEMENT described in this report on 5-28, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-3-97 Inspector: Robert T Kelley Commissions: IL932, NB7742NISB
(State or Province, National Board)

UNIT 3 WEST [illegible]

FORM U-1 MANUFACTURERS' DATA REPORT FOR UNFIRED PRESSURE VESSELS

As required by the Provisions of the ASME Code Rules

1. Manufactured by Berlin Chapman, Div. Perfex Corp., Berlin, Wisconsin (Name and address of Manufacturer)

2. Manufactured for General Electric Company, San Jose, California (Name and address of Purchaser)

3. Type Vertical (Horiz. or Vert.) Kind Heat Exch. (Tank, Jacketed, Heat Exch.) Vessel No. 05036-3 (Mfrs. Serial) ( ) (State & State No.) Natl. Bd. No. 3006 Yr. Built 1967

Items 4-9 incl. to be completed for ~~single wall vessels, jackets of jacketed vessels~~ or shells of heat exchangers.

4. SHELL: Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Fig. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 21 Ft. 4 1/16

5. SEAMS: Long Dbl. Weld Butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl. Weld Butt H.T. No X.R. Complete Sectioned No No. of Courses 3

If riveted describe seams fully on reverse side of form.

6. HEADS (a) Material See Item 10 T.S. (b) Material T.S.

Location (Top, bottom, ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a)								
(b)								

If removable, bolts used (Material, Spec. No., T.S., Size, Number) Other fastening (Describe or Attach Sketch)

7. STAYBOLTS: (Material) If hollow (Size of Hole) Attachment (Threaded, Welded) Pitch (Horiz.) X (Vert.) Diam. (Nominal)

8. JACKET CLOSURE: (Describe as ogee & weld, bar, etc. If bar, give dimensions, if bolted, describe or sketch)

9. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than −20°) °F. Hydrostatic ~~Pneumatic~~ ~~Combination~~ Test Press. 563 psi.

Items 10 and 11 to be completed for tube sections.

10. TUBE SHEETS: Stationary. Material SA-212-B FBX (Kind & Spec. No.) Diam. 62 3/4 In. (Subject to Pressure) Thickness 3 3/4 In. Attachment Welded (Welded, Bolted)

(2)

Floating. Material (Kind & Spec. No.) Diam. In. Thickness In. Attachment

70-30 CU.NI.

11. TUBES: Material SB-111 (Kind & Spec. No.) O.D. 3/4 In. Thickness 18 BWG Inches or Gage Number 2512 Type Straight (Straight or U)

Items 12-15 incl. to be completed for ~~inner chambers of jacketed vessels, or~~ channels of heat exchangers. (2)

12. SHELL Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Fig. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 2 - 3 1/2 / 1 Ft. 11 3/4

13. SEAMS: Long Dbl. Weld butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl. Weld butt H.T. No X.R. Complete Sectioned No No. of courses 1

If riveted describe seams fully on reverse side of form.

14. HEADS (a) Material T.S. (b) Material SA-212-B T.S. 70000 (c) Material SA-212-B T.S. 70000

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a) Top, bottom, ends								
(b) Channel Top	8"						70 1/8"	
(c) ~~Floating~~ Bot.	5 3/4"						70 1/8"	

If removable, bolts used (a) (Material, Spec. No., T.S., Size, Number) (b) 60 − 1 3/8" SA-193 B7 125000

(c) 60 − 1 3/8" SA-193 B7 125000 Other fastening (Describe or Attach Sketch)

15. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than −20°) °F. Hydrostatic ~~Pneumatic or~~ ~~Combination~~ Test Press. 563 psi.

Items below to be completed for all vessels where applicable.

16. SAFETY VALVE OUTLETS: Number Size Location

17. NOZZLES

Purpose (Inlet, Outlet, Drain)	Number	Diam. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
Inlet-Outlet	2	24"x18"	Con. Red. Weld End	SA-106-B	.500"	SA-212-B	Welded
Inlet-Outlet	2	16"	Weld End	SA-106-B	.500"	SA-212-B	Welded
Vent	3	1"	Socket Weld	SA-181-1	6000#	Coupling	Welded
Drain	2	2"	Socket Weld	SA-181-1	6000#	Coupling	Welded

*If postweld heat treated. ** List other internal or external pressures with coincident temperature when applicable.

FORM U-1 (back)

18. INSPECTION OPENINGS: Manholes, No. ______ Size ______ Location ______
Handholes, No. ______ Size ______ Location Removable Heads
Threaded, No. ______ Size ______ Location ______

19. SUPPORTS: Skirt No (Yes or No) Lugs ______ (Number) Legs ______ (Number) Other Brackets (Describe) Attached Weld to Shell (Where & How)

20. REMARKS: Containment Exchanger Item 1
Shell side - Demineralized water - Tube side - River water
Channel side of tube sheets ⅛ Monel Overlay

Channel Flgs. SA-105-2 5 5/16" tk.

(Brief description of purpose of the vessel, as Air Tank, After Cooler, Jacketed Cooker, etc. State contents of each part.)

We certify that the statements made in this report are correct and that all details of design, material, construction, and workmanship of this vessel conform to the ASME Code for Unfired Pressure Vessels.

Date AUG 23 1967 19 ______ Signed Berlin Chapman (Manufacturer) By W. A. Zimmer

Certificate of Authorization Expires December 31, 1970

CERTIFICATE OF SHOP INSPECTION

VESSEL MADE BY Berlin Chapman at Berlin, Wisconsin

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of N.B. and employed by Hartford Steam Boiler I. & I. Co. of Hartford, Conn. have inspected the pressure vessel described in this manufacturer's data report on AUG 23 1967 19 ___, and state that to the best of my knowledge and belief, the manufacturer has constructed this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date AUG 23 1967

[signature] Commissions N.B. 1492
Inspectors Signature Nat'l Board or State and No.

CERTIFICATE OF FIELD ASSEMBLY INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of ______ and employed by ______ of ______ have compared the statements in this manufacturer's data report with the described pressure vessel and state that parts referred to as data items ______, not included in the certificate of shop inspection have been inspected by me and that to the best of my knowledge and belief the manufacturer has constructed and assembled this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code. The described vessel was inspected and subjected to a hydrostatic test of ______ psi.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date ______ 19 ______

______ Commissions ______
Inspector's Signature Nat'l Board or State and No.

Printed in U.S.A. (7/66) This Form is obtainable from the ASME, 345 E. 47th St., New York, N.Y. 10017

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2/3

3. Work Performed By: ComEd (Name)
Same as Above (Address)

WR 960011774-02 (PLAN 3-97-004)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 68 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6266	N/A	None	N/A	Repair	No

7. Description of work: Repaired spare main steam safety valve obtained from Oyster Creek Nuclear Power Plant. As part of receipt inspection, Dresden Mechanical Maintenance Department performed a rebuild of the valve. A linear indication was discovered in the discharge surface of the valve body. Indication was removed with light grinding.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve returned to Stores as spare assembly.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR** described in this report on 6-13, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-13-97 Inspector: Kurt T Rahney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-5-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Bechtel Constructors (Name) WR 960096685 (PLAN 3-97-006)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1600 Containment (Torus)

5. (a) Construction Code ASME Section III, 19 65/77 Edition, S65/S77 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 92 Edition, 92 Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Additional Plates for Torus Penetration X-303A Nozzle Reinforcement	Unknown	Heat # 350D34520	N/A	SI #814B86	N/A	Replacement	No
Additional Plates for Torus Penetration X-303B Nozzle Reinforcement	Unknown	Heat # 350D34520	N/A	SI #814B86	N/A	Replacement	No
Additional Plates for Torus Penetration X-303C Nozzle Reinforcement	Unknown	Heat # 350D34520	N/A	SI #814B86	N/A	Replacement	No
Additional Plates for Torus Penetration X-303D Nozzle Reinforcement	Unknown	Heat # 350D34520	N/A	SI #814B86	N/A	Replacement	No
Torus ladder attachment welds	Unknown	N/A	N/A	N/A	N/A	Replacement	No

7. Description of work: Added eight reinforcement plates (1¼" X 2½" X 5" SA516 Grade 70) to existing Unit 3 Torus Emergency Core Cooling System (ECCS) penetrations as part of Modification M12-3-96-006 which installed new ECCS suction strainers in response to NRC Bulletin 96-03. Existing torus ladder attachments were removed in order to get new strainers into the torus manway and were rewelded into place after all four strainers were installed into torus.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-16 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-16, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-16-97 Inspector: Kurt T. Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-28-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

Same as Above (Address)

WR 950063467 (PLAN 3-97-007)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0/ASME Section VIII, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" Relief Valve	Dresser Industries	UNKNOWN	N/A	3-1501-17B	N/A	Replaced	No
2" Relief Valve	Dresser Industries	TH-88701	N/A	SI #503E01	N/A	Replacement	No

7. Description of work: Replaced existing "B" LPCI discharge relief valve which failed IST surveillance with brand new valve.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 165 psig Test Temperature 69 °F

9. Remarks: Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-29 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-11, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: Kurt T. Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-28-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

Same as Above (Address)

WR 950063481 (PLAN 3-97-008)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0/ASME Section VIII, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
1½" Relief Valve	Dresser Industries	UNKNOWN	N/A	3-1501-13B	N/A	Replaced	No
1½" Relief Valve	Dresser Industries	TL-11444	N/A	SI #814A24	N/A	Replacement	No

7. Description of work: Replaced existing suction relief valve for "B" LPCI pump which failed IST surveillance with brand new valve.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 4.4 psig Test Temperature 61 °F

9. Remarks: Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-29 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 5-30, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-30-97 Inspector: Curt T Lowery Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd (Name)
Same as Above (Address)

WR 940096467 (PLAN 3-97-009)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0205 Reactor Head Spray

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2½" Dual Disc Check Valve for Reactor Head Spray	Mission	Unknown	N/A	3-0205-27	N/A	Replaced	No
2½" Nozzle Check Valve for Reactor Head Spray	Anchor-Darling	EZ409-3-1	N/A	SI #815G23	1996	Replacement	No

7. Description of work: Replaced existing dual disc check valve assembly to address long standing seat leakage problems. New valve is a nozzle check valve. New valve was made in accordance with ASME Section III 1989 Edition (less N-Stamp). Work performed per alternate replacement D-1996-011-00-A.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1050 psig Test Temperature 200 °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-11 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-12, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-12-97 Inspector: Robt J Carney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-4-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WR 960116198 (PLAN 3-97-010)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 HPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Heat Exchanger Bolting (1" -11 Hex Nuts)	UNKNOWN	UNKNOWN	N/A	3-2303-SOC	N/A	Replaced	No
Heat Exchanger Bolting (1" -11 Bolts)	UNKNOWN	UNKNOWN	N/A	3-2303-SOC	N/A	Replaced	No
1" -11 Hex Nuts	UNKNOWN	NONE	N/A	SI #796C99	N/A	Replacement	No
1" -11 Bolts	UNKNOWN	NONE	N/A	SI #768G96	N/A	Replacement	No

7. Description of work: Replaced existing Unit 3 HPCI Lube Oil Cooler head bolting with new material to accommodate installation of hardened washers.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Mechanical Maintenance performed leak check after maintenance was completed.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-4 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-11, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: Kent T. Reinert Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-23-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 2/3

3. Work Performed By: Same as Above (Name)
Same as Above (Address)

WR 970017094-03 (PLAN 3-97-011)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 68 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6265	N/A	None	N/A	Repair	No

7. Description of work: Valve was obtained from Oyster Creek Power Plant. Valve was rebuilt by Dresden Station Mechanical Maintenance Department as part of receipt inspection. Casting flaws observed during VT-3/4 inspection of valve internals. As conservative measure, flaws were removed per valve OEM recommendations. Valve reassembled and returned to Stores as spare assembly.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-23 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR** described in this report on 5-23, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-23-97 Inspector: Robt T Lowery Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-23-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 2/3

3. Work Performed By: Same as Above (Name) WR 970017094-04 (PLAN 3-97-012)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0203 Main Steam

5. (a) Construction Code ASME Section III, 19 68 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
6" Main Steam Safety Valve	Consolidated/Dresser	BK 6270	N/A	None	N/A	Repair	No

7. Description of work: Valve was obtained from Oyster Creek Power Plant. Valve was rebuilt by Dresden Station Mechanical Maintenance Department as part of receipt inspection. Casting flaws observed during VT-3/4 inspection of valve internals. As conservative measure, flaws were removed per valve OEM recommendations. Valve reassembled and returned to Stores as spare assembly.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-23 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR described in this report on 5-23, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-23-97 Inspector: Robert T Kenney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7-2-97

Sheet: 1 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)

Same as Above (Address)

WR 970013877 (PLAN 3-97-013)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 2300 HPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases N416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-1187D-121	Unknown	None	N/A	None	N/A	Replaced	No
Support M-1187D-122	Unknown	None	N/A	None	N/A	Replaced	No
Support M-1187D-123	Unknown	None	N/A	None	N/A	Replaced	No
Line 3-2318-2"-LX	Unknown	None	N/A	None	N/A	Replaced	No
¼" A36 Plate (for Supports)	Unknown	Heat # K50262	N/A	SI #781D43	N/A	Replacement	No
½" A36 Plate (for Supports)	Unknown	Heat # 65376	N/A	SI #779G69	N/A	Replacement	No

7. Description of work: Replaced material on existing HPCI Gland Seal Leak Off Line supports and piping as part of upgrade of system from Non-Safety Related to Safety-Related. System was also classified to Section XI Class 2. All work performed under E12-3-97-201.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure Nominal psig Test Temperature Nominal °F

9. Remarks: Piping was given VT-2 examination during HPCI surveillance DOS 23-003 on 6/16/97..

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 7-2 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 7-3, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 7-3-97 Inspector: Robert T Rainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

COPY

CATEGORY 3

FORM NIS-2 SUPPLEMENT
OWNER'S REPORT OF REPAIR OR REPLACEMENT SUPPLEMENTAL SHEET

DAP 11-18
REVISION 07

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 7-2-97

Sheet: 2 Of 2

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Construction (Name)

WR 970013877 (PLAN 3-97-013)
Repair Organization P.O. No., Job No. etc.

Same as Above (Address)

4. Identification of System: 2300 HPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Case N416-1

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
2" Pipe Strap	Grinnell	Part Number CL144N	N/A	SI #819B04	N/A	Replacement	No
3" X 3" X ¼" A500 Grade B Tube Steel	Unknown	Heat # 862243	N/A	SI #772H10	N/A	Replacement	No
2" Schedule 80 A106 Grade B Pipe	Unknown	Heat # B45687	N/A	SI #799H20	N/A	Replacement	No
2" 90 Degree A105 Socket Weld Elbows	Unknown	Heat # 37627	N/A	SI #558A69	N/A	Replacement	No
2" 45 Degree A105 Socket Weld Elbows	Unknown	Heat # AA215	N/A	SI #500B95	N/A	Replacement	No
2" X 1" A105 Socket Weld Reducer	Unknown	Heat # 59119	N/A	SI #799D53	N/A	Replacement	No

COPY

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-27-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: ComEd/Bechtel (Name) WR 940096861 (PLAN 3-97-014)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1100 Standby Liquid Control

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3-1101-15 (Standby Liquid Control Inboard Check Valve)	Crane Valve	None	N/A	3-1101-15	N/A	Repair	No

7. Description of work: Valve failed local leak rate. Valve bonnet is seal welded, in ordet to gain access to internals the seal weld must be ground off. After maintenance was completed on valve internals, bonnet was reinstalled and seal welded again.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 73.2 °F

9. Remarks: Valve was inspected during Standby Liquid Control injection test and surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-27 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR** described in this report on 5-24, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-29-97 Inspector: Kurt T. Rowe Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-28-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

Same as Above (Address)

WR 940096861 (PLAN 3-97-017)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 CCSW

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
10" Dual Disc Check Valve	Unknown	Unknown	N/A	3-1501-1B	N/A	Replaced	No
1"-8 A193 Grade B7 Bolts	Unknown	Unknown	N/A	None	N/A	Replaced	No
1"-8 A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	None	N/A	Replaced	No
10" Dual Disc Check Valve	Gulf Valve Company	29206-1-1	N/A	SI #814B60	N/A	Replacement	
1"-8 A193 Grade B7 Bolts	Cardinal Industries	Unknown	N/A	SI #815H20	N/A	Replacement	No
1"-8 A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	SI #796D05	N/A	Replacement	No

7. Description of work: Replaced existing pump discharge check valve (dual disc check valve) and associated flange bolting. Check valve changed under station check valve program and flange bolting replaced with new material in order to accommodate the addition of hardened washers.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 225 psig Test Temperature 67 °F

9. Remarks: Examination performed during CCSW pump run.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-3 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-11, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: Kurt T Lewis Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-7-97
One First National Plaza, Chicago IL, 60690 (Address)

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Bechel Constructors (Name) WR 960036553 (PLAN 3-97-018)
Repair Organization P.O. No., Job No. etc.
Gaithersberg, MD 20877 (Address)

4. Identification of System: 0201 Reactor Vessel

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Reactor Head-to-Flange weld (Segment "D")	Babcock & Wilcox	610-0111-51	N-139	3-THD-FLGD	1969	Repair	Yes
Reactor Head-to-Flange weld (Segment "E")	Babcock & Wilcox	610-0111-51	N-139	3-THD-FLGE	1969	Repair	Yes
Reactor Head-to-Flange weld (Segment "F")	Babcock & Wilcox	610-0111-51	N-139	3-THD-FLGF	1969	Repair	Yes

7. Description of work: During dry powder magnetic particle examination for ISI, linear indications were found in three of the six segments of the reactor vessel upper head to flange weld. None of the Indications were deep (< .0625") and were removed with minor grinding. The longest indication was 0.7" in length.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [ ]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Indications were removed with very minor grinding and removal was verified with magnetic particle inspection. A PIF was generated in accordance with site procedures. The other segments (3-THD-FLGA, 3-THD-FLGB and 3-THD-FLGC were examined as part of the original ISI scope and were free of recordable indications.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-7 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPLACEMENT** described in this report on 5-9, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-9-97 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

SHEET 1 OF 3

2-8-71
REV. #2
610-0111

FORM N-1A MANUFACTURERS' DATA REPORT FOR NUCLEAR VESSELS

Alternate Form for Single Chamber Completely Shop-Fabricated Vessels Only

As required by the Provisions of the ASME Code Rules

1. Manufactured by Babcock & Wilcox Company Mt. Vernon, Indiana
(Name and address of Manufacturer)

2. Manufactured for General Electric - APED Dresden III Morris, Illinois
(Name and address of Purchaser)

3. Type Reactor (Horiz. or Vert.) Vessel No. (610-0111-51-52) (Mfrs. Serial) (State or State No.) Natl. Bd. No. N-139 Year Built 1969

4. Shell: Material SA302B1339 (Kind & Spec. No.) T.S. 80,000 (Min. of range specified) Thk. 6.125" min. Corr. Allow 1/16" Diam. 20' 11 3/8" Length 69' 8"

5. Seams: Long butt welded H.T.[1] yes X.R. yes Efficiency ____ % (If Class B)

Girth butt welded H.T.[1] yes X.R. yes No. of Courses 4

6. Heads: (a) Material SA302B1339 T.S. 80,000 (b) Material SA302B1339 T.S. 80.000

Location (Top, bottom, ends)	min. Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a) top	4"					125 11/16" IR		concave
(b) bottom	6 1/8" & 8"					125 11/16" IR		concave

Lower Closure

If removable, bolts used SA193-1335P4 (Material, Spec. No., T.S., Size, Number) 145,000 TS min. 6" dia. 92 (Describe or Attach Sketch)

7. Constructed for operating press [2] 1250 psi at max. temp 575 °F. at temp. of +40 °F. Charpy impact 35 ft-lb Hydrostatic ~~Pneumatic or Combination~~ Test Press 1565 psi @100°F.

8. Safety or Relief Valve Outlets: Number none Size -- Location --

9. Nozzles:

Purpose (Inlet, Outlet, Drain)	Number	Diam. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
See Supplemental Sheet #2							

10. Inspection Manholes, No. none Size ____ Location ____

Openings: Handholes, No. none Size ____ Location ____

Threaded, No. none Size ____ Location ____

Supports: Skirt yes (Yes or No) Lugs - (Number) Legs - (Number) Other - (Describe) Attached welded to lower head (Where & How)

12. Remarks A. Class A Reactor Vessel - Contents: water.
B. Vessel clad internally with 1/8" min. SA-371 ER308L except portions of lower head are clad with 1/8" min. inconel SB304-ERNICR-3.
C. See Supplemental Sheet #2 for shell flange and closure head flange material and size.
D. See Supplemental Sheet #3 for revisions.
(Brief description of purpose of the vessel—State Contents.)

[1] If Postweld Heat-Treated.
[2] List other internal or external pressure with coincident temperature when applicable.

We certify that the statements made in this report are correct and that all details of material design, construction, and workmanship of this vessel conform to the ASME Code for Nuclear Vessels.

Date February 8 19 71 Signed Babcock & Wilcox Company (Manufacturer) By [signature] Project Engineer

Rev. #2

Certificate of Authorization Expires April 10, 1972 NPGD Component Engineering

CERTIFICATION OF DESIGN

Design information on file at Babcock & Wilcox Company Barberton, Ohio

Stress analysis report on file at Babcock & Wilcox Company Barberton, Ohio

Design specifications certified by R.L. Call Prof. Eng. yes State Cal. Reg. No. 13540

Stress analysis report certified by J.P. Butti Prof. Eng. yes State Ohio Reg. No. E29810

CERTIFICATE OF SHOP INSPECTION

VESSEL MADE BY Babcock & Wilcox Company at Mt. Vernon, Indiana

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of ____ and employed by Hartford S.B.I. & I. Co. of Hartford, Connecticut

have inspected the pressure vessel described in this manufacturer's data report on December 16 1969, and state that to the best of my knowledge and belief, the manufacturer has constructed this pressure vessel in accordance with the ASME Code for Nuclear Vessels.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 2-11 19 71

R. L. Watson
Inspector's Signature

Commissions NB #3864
Nat'l Board, State or Province and No.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-3-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Same as Above (Name) WR 970044798 (PLAN 3-97-020)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
24" X 14" Field Fabricated Tee (Lines 3-1507-24"-DX and 3-1507B-14"-DX)	N/A	N/A	N/A	Weld 24-11	N/A	Repair	No
5/16" A516 Grade 70 Plate	Unknown	Heat #23724	N/A	SI #819E48	N/A	Replacement	No
5/16" A516 Grade 70 Plate	Unknown	Heat #23724	N/A	SI #819F48	N/A	Replacement	No

7. Description of work: Reinforced existing 24" X 14" field fabricated tee per E12-3-97-219 in; order to support licensing ammendment on CCSW. SI #819E48 plate rolled for 24" pipe and SI #819F48 is plate rolled for 14" pipe.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 7.0 psig Test Temperature Ambient °F

9. Remarks: Examination performed with torus static head pressure. Pressure taken at 3B LPCI pump suction.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-3 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR/REPLACEMENT described in this report on 6-3, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-3-97 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-3-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: ComEd/Bechtel (Name) WR 950063479 (PLAN 3-97-021)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0ASME Section VIII, 19 67/65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
1½" Relief Valve	Dresser Industries	UNKNOWN	N/A	3-1501-13C	N/A	Replaced	No
1½" Relief Valve	Dresser Industries	TL-11448	N/A	SI #814A24	N/A	Replacement	No

7. Description of work: Replaced existing suction relief valve which failed IST surveillance with brand new valve.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 4.5 psig Test Temperature 68 °F

9. Remarks: Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-3 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-4, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-4-97 Inspector: Robert J. Kinney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-28-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Same as Above (Name) WR 950063478 (PLAN 3-97-022)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
1½" Relief Valve	Dresser Industries	UNKNOWN	N/A	3-1599-13D	N/A	Replaced	No
1½" Relief Valve	Dresser Industries	TL-11446	N/A	SI #814A24	1991	Replacement	No

7. Description of work: Replaced existing suction relief valve which failed IST surveillance with brand new valve.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 4.5 psig Test Temperature 68 °F

9. Remarks: Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-28 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 4/30-, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-30-97 Inspector: Kurt T Gowey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-24-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Same as Above (Name) WR 950061006 (PLAN 3-97-025)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1100 Standby Liquid Control

5. (a) Construction Code USAS B31.1.0/ASME Section VIII, 19 67/65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
1 ½" X 2" Relief Valve	Unknown	Not Recorded	N/A	3-1105B	N/A	REPLACED	No
5/8" Diameter A193 B7 Bolting	Crosby	None	N/A	3-1105B	N/A	REPLACED	No
1 ½" X 2" Relief Valve	Crosby	Not Recorded	N/A	SI #814B60	N/A	REPLACEMENT	No
5/8" Diameter A193 B7 Bolting	Unknown	None	N/A	SI #796D73	N/A	REPLACEMENT	No

7. Description of work: Replaced existing relief valve which failed IST surveillance. Existing bolting was replaced in ordert to achieve full nut engagement due to addition of hardened washers. Relief that was removed will be rebuilt and returned to Stores as spare.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040/1330 psig Test Temperature 73/Ambient °F

9. Remarks: VT-2 of valve perfromed during Standby Liquid Control injection test and during IST surveillance of "B" Standby Liquid Control pump.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-24 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 5-27, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-27-97 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-17-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: ComEd\Bechtel (Name) WR 950060516 (PLAN 3-97-027)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1500 LPCI/CCSW

5. (a) Construction Code ASME Section III, 19 65 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3B CCSW/LPCI Heat Exchanger	Berlin-Chapman	05036-3	3006	3B-1503	1967	Repair	Yes

7. Description of work: Repaired pitted areas in lower channel of heat exchanger by weld build-up.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 175/165 psig Test Temperature 58/68 °F

9. Remarks: Test pressure and test temperature are tube side and shell side respectively. Examination performed during LPCI surveillance.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed: Brendan J Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-17 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connectictu having inspected the **REPAIR** described in this report on 6-19, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-19-97 Inspector: Kurt T Carney Commissions: IL932, NB7742NISB
(State or Province, National Board)

UNIT 3 WEST "B" LOOP

FORM U-1 MANUFACTURERS' DATA REPORT FOR UNFIRED PRESSURE VESSELS

As required by the Provisions of the ASME Code Rules

1. Manufactured by Berlin Chapman, Div. Perfex Corp., Berlin, Wisconsin (Name and address of Manufacturer)

2. Manufactured for General Electric Company, San Jose, California (Name and address of Purchaser)

3. Type Vertical (Horiz. or Vert.) Kind Heat Exch. (Tank, Jacketed, Heat Exch.) Vessel No. 05036-3 (Mfrs. Serial) ( ) (State & State No.) Natl. Bd. No. 3006 Yr. Built 1967

Items 4-9 incl. to be completed for ~~single wall vessels (such as air tanks), jackets of jacketed vessels or~~ shells of heat exchangers.

4. SHELL: Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Flg. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 21 Ft. 4 5/16

5. SEAMS: Long Dbl. Weld Butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl. Weld Butt H.T. No X.R. Complete Sectioned No No. of Courses 9

If riveted describe seams fully on reverse side of form.

6. HEADS (a) Material See Item 10 T.S. ___ (b) Material ___ T.S. ___

Location (Top, bottom, ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a)								
(b)								

If removable, bolts used ___ (Material, Spec. No., T.S., Size, Number) Other fastening ___ (Describe or Attach Sketch)

7. STAYBOLTS: ___ (Material) If hollow ___ (Size of Hole) Attachment ___ (Threaded, Welded) Pitch ___ (Horiz.) X ___ (Vert.) Diam. ___ (Nominal)

8. JACKET CLOSURE: ___ (Describe as ogee & weld, bar, etc. If bar, give dimensions, if bolted, describe or sketch)

9. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than −20°) ___ °F. Hydrostatic ~~Pneumatic or Combination~~ Test Press. 563 psi.

Items 10 and 11 to be completed for tube sections.

10. TUBE SHEETS: Stationary. Material SA-212-B FBX (Kind & Spec. No.) Diam. 62 3/4 In. (Subject to Pressure) Thickness 3 3/4 In. Attachment Welded (Welded, Bolted)

(2)

Floating. Material ___ (Kind & Spec. No.) Diam. ___ In. Thickness ___ In. Attachment ___

11. TUBES: Material 70-30 CU.NI. SB-111 (Kind & Spec. No.) O.D. 3/4 In. Thickness 18 BWG Inches or Gage Number 2512 Type Straight (Straight or U)

Items 12-15 incl. to be completed for ~~inner chambers of jacketed vessels, or~~ channels of heat exchangers. (2)

12. SHELL Material SA-212-B (Kind and Spec. No.) T.S. FBX 70000 (Flg. or F.B. & Spec. Min. T.S.) Nominal Thickness 7/8 In. Corrosion Allowance 1/8 In. Diam. 5 Ft. 1 In. Length 1 Ft. 11 3/4 (2 - 3 1/2)

13. SEAMS: Long Dbl. Weld butt (Welded, Dbl., Single, Lap, Butt) H.T. No (Yes or No) X.R. Complete (Spot or Complete) Sectioned No (Yes or No) Efficiency 100 %

Girth Dbl. Weld butt H.T. No X.R. Complete Sectioned No No. of courses 1

If riveted describe seams fully on reverse side of form.

14. HEADS (a) Material ___ T.S. ___ (b) Material SA-212-B T.S. 70000 (c) Material SA-212-B T.S. 70000

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Pressure (Convex or Concave)
(a) Top, bottom, ends								
(b) Channel Top	8"						70 1/8"	
(c) ~~Floating~~ Bot.	5 3/4"						70 1/8"	

If removable, bolts used (a) ___ (Material, Spec. No., T.S., Size, Number) (b) 60 - 1 3/8" SA-193 B7 125000

(c) 60 - 1 3/8" SA-193 B7 125000 Other fastening ___ (Describe or Attach Sketch)

15. Constructed for max. allowable working press. 375 psi at max. temp. 281 °F. Min. temp. (when less than −20°) ___ °F. Hydrostatic ~~Pneumatic or Combination~~ Test Press. 563 psi.

Items below to be completed for all vessels ~~where~~ applicable.

16. SAFETY VALVE OUTLETS: Number ___ Size ___ Location ___

17. NOZZLES

Purpose (Inlet, Outlet, Drain)	Number	Diam. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
Inlet-Outlet	2	24"x18"	Con. Red. Weld End	SA-106-B	.500"	SA-212-B	Welded
Inlet-Outlet	2	16"	Weld End	SA-106-B	.500"	SA-212-B	Welded
Vent	3	1"	Socket Weld	SA-181-1	6000#	Coupling	Welded
Drain	2	2"	Socket Weld	SA-181-1	6000#	Coupling	Welded

*If postweld heat treated. **List other internal or external pressure with coincident temperature when applicable.

FORM U-1 (back)

18. INSPECTION OPENINGS: Manholes, No. ____ Size ____ Location ____
Handholes, No. ____ Size ____ Location Removable Heads
Threaded, No. ____ Size ____ Location ____

19. SUPPORTS: Skirt No (Yes or No) Lugs ____ (Number) Legs ____ (Number) Other Brackets (Describe) Attached Weld to Shell (Where & How)

20. REMARKS: Containment Exchanger Item 1
Shell side - Demineralized water - Tube side - River water
Channel side of tube sheets ½ Monel Overlay

Channel Flgs. SA-105-2 5 5/16" tk.

(Brief description of purpose of the vessel, as Air Tank, After Cooler, Jacketed Cooker, etc. State contents of each part.)

We certify that the statements made in this report are correct and that all details of design, material, construction, and workmanship of this vessel conform to the ASME Code for Unfired Pressure Vessels.

Date AUG 2 3 1967 19____ Signed Berlin Chapman (Manufacturer) By [signature]

Certificate of Authorization Expires December 31, 1970

CERTIFICATE OF SHOP INSPECTION

VESSEL MADE BY Berlin Chapman at Berlin, Wisconsin

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of N.B. and employed by Hartford Steam Boiler I. & I. Co. of Hartford, Conn. have inspected the pressure vessel described in this manufacturer's data report on AUG 2 3 1967 19___, and state that to the best of my knowledge and belief, the manufacturer has constructed this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date AUG 2 3 1967 19____

[signature] Inspectors Signature Commissions N.B. 1492 Nat'l Board or State and No.

CERTIFICATE OF FIELD ASSEMBLY INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State of ____ and employed by ____ of ____ have compared the statements in this manufacturer's data report with the described pressure vessel and state that parts referred to as data items ____ not included in the certificate of shop inspection have been inspected by me and that to the best of my knowledge and belief the manufacturer has constructed and assembled this pressure vessel in accordance with the applicable sections of the ASME Boiler and Pressure Vessel Code. The described vessel was inspected and subjected to a hydrostatic test of ____ psi.

By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the pressure vessel described in this manufacturer's data report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date ____ 19____

____ Inspector's Signature Commissions ____ Nat'l Board or State and No.

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-28-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)

Same as Above (Address)

WR 970048050 (PLAN 3-97-029)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1300 Isolation Condenesr

5. (a) Construction Code AISC, 19 6th Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Flued Head Anchor at Penetration X-108A	Unknown	N/A	N/A	X-108A	N/A	Repair	No
½" A36 Plate	Unknown	Heat A1653	N/A	SI #779B98	N/A	Replacement	No
¼" A36 Plate	Unknown	Heat N997	N/A	SI #770D00	N/A	Replacement	No

7. Description of work: Reinforced existing flued head anchor by increasing fillet weld sizes and adding additional reinforcement plates per E12-3-97-221.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-28 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 5-28, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-28-97 Inspector: Robert Reed Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-22-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name) Unit: 3
6500 North Dresden Road, Morris IL., 60450 (Address)

3. Work Performed By: Same as Above (Name) WR 970052950 (PLAN 3-97-030)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 0220/1600 Feedwater System Flued Head Anchor

5. (a) Construction Code AISC, 19 6th Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Penetration X-107B	Unknown	None	N/A	X-107B	N/A	Repair	NO

7. Description of work: Coped section of existing horizontal stiffener plate to eliminate interference. Existing configuration was contacting the 2D main steam isolation valve bonnet when piping heated up. By coping section of stiffener, future interference has been eliminated.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: VT-3/4 inspection performed after horizontal stiffener was coped.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-22 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR** described in this report on 5-22, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-22-97 Inspector: [signature] Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 6-12-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: ComEd/Bechtel (Name) WR 960001532 (PLAN 3-97-031)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1200 Reactor Clean Up

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Anchor-Darling Dual Disc Gate Valve Disc for 3-1201-1 Valve	Anchor-Darling	Unknown	N/A	3-1201-1	N/A	Replaced	No
Anchor-Darling Dual Disc Gate Valve Disc for 3-1201-1 Valve	Anchor-Darling	Unknown	N/A	SI #816A80	N/A	Replacement	No

7. Description of work: Replaced existing valve disc which was outside of vendor allowed tolerances with new disc assembly. Exisitng valve disc had failed local leak rate test.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Valve was examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 6-12, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-12-97 Inspector: Kurt T. Kaiser Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-3-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Bechtel Constructors (Name)
Same as Above (Address)

WR 970052619 (PLAN 3-97-033)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1500 LPCI

5. (a) Construction Code USAS B31.1.0ASME Section III , 19 67/77 Edition, NO/S77 Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-3414-11	N/A	N/A	N/A	M-3414-11	N/A	Repair	No
Shear Pins (1½" A36 round stock)	Unknown	Heat # 559712716	N/A	SI #800H41	N/A	Replacement	No

7. Description of work: Modified existing support to restore design margin in 3B LPCI heat exchanger bypass line (3-1534-18"-D). Work performed in accordanc with E12-3-97-223.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Support modified by adding shear pins into baseplate.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPAIR/REPLACEMENT Conforms to Section XI of the ASME Code.

Signed : Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-10, 1997 (Date)

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPAIR/REPLACEMENT described in this report on 6-11, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-11-97 Inspector: [signature] Commissions: IL932, NB7742NISB (State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-22-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

Same as Above (Address)

WR 960032654 (PLAN 3-97-035)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 6600 Diesel Generator

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Bolting (J-429 Grade B5)	UNKNOWN	UNKNOWN	N/A	2/3-6699A	N/A	REPLACED	NO
Hex Nuts (A-194 Grade 2H)	UNKNOWN	UNKNOWN	N/A	2/3-6699A	N/A	REPLACED	NO
Bolting (A-193 Grade B7)	UNKNOWN	UNKNOWN	N/A	SI #500E52	N/A	REPLACEMENT	NO
Hex Nuts (A-194 Grade 2H)	UNKNOWN	Heat Code DJQ	N/A	SI #796C99	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing head bolting on "A" diesel generator cooling water heat exchanger due to incorrect bolting material (Grade B5 instead of Grade B7) and in order to add longer bolting to achieve full nut engagement due to the addition of hardened washers.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 30 psig Test Temperature 83 °F

9. Remarks: Pressure and temperature taken at heat exchanger outlet during DOS 6600-01 (diesel generator surveillance).

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-27 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 5-28, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-28-97 Inspector: Kurt T. Kaney Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 5-22-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: Same as Above (Name)

Same as Above (Address)

WR 960032651 (PLAN 3-97-036)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 6600 Diesel Generator

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Bolting (J-429 Grade B5)	UNKNOWN	UNKNOWN	N/A	2/3-6699B	N/A	REPLACED	NO
Hex Nuts (A-194 Grade 2H)	UNKNOWN	UNKNOWN	N/A	2/3-6699B	N/A	REPLACED	NO
Bolting (A-193 Grade B7)	UNKNOWN	UNKNOWN	N/A	SI #500E52	N/A	REPLACEMENT	NO
Hex Nuts (A-194 Grade 2H)	UNKNOWN	Heat Code DJQ	N/A	SI #796C99	N/A	REPLACEMENT	NO

7. Description of work: Replaced existing head bolting on "B" diesel generator cooling water heat exchanger due to incorrect bolting material (Grade B5 instead of Grade B7) and in order to add longer bolting to achieve full nut engagement due to the addition of hardened washers.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 31 psig Test Temperature 83 °F

9. Remarks: Pressure and temperature taken at heat exchanger outlet during DOS 6600-01 (diesel generator surveillance).

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-27 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPLACEMENT** described in this report on 5-24, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-29-97 Inspector: Robert T. Lavine Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-11-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: ComEd/Bechtel (Name)

Same as Above (Address)

WR 970057096 (PLAN 3-97-037)
Repair Organization P.O. No., Job No. etc.

4. Identification of System: 1400 Core Spray

5. (a) Construction Code AISC, 19 6TH Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
Support M-3409-19	N/A	N/A	N/A	M-3409-19	N/A	Repair	No
A36 Plate (New)	Unknown	Heat Number 2B9391	N/A	SI #779B98	N/A	Replacement	No

7. Description of work: Modified existing support by adding stiffener plate between existing baseplate and tube steel. Added fillet welds on the remaining sides of existing tube steel and baseplate. Work performed per E12-3-97-224. Support was modified to allow increase of torus bulk temperature.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: None.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR/REPLACEMENT** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-12 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR/REPLACEMENT** described in this report on 6-12, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-12-97 Inspector: Robert T Gainey Commissions: IL932, NB7742NISB
(State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name)
One First National Plaza, Chicago IL, 60690 (Address)

Date: 6-16-97

Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address)

Unit: 3

3. Work Performed By: General Electric (Name)

WR 950070767 (PLAN 3-97-038)
Repair Organization P.O. No., Job No. etc.

Same as Above (Address)

4. Identification of System: 0215/0300 Reactor Head Vent/Reactor Head Spray

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
A193 Grade B7 Bolts	Unknown	Unknown	N/A	N8 Nozzle	N/A	Replaced	No
A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	N8 Nozzle	N/A	Replaced	No
A193 Grade B7 Bolts	Unknown	Unknown	N/A	N18 Nozzle	N/A	Replaced	No
A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	N18 Nozzle	N/A	Replaced	No
A193 Grade B7 Bolts	Unknown	Unknown	N/A	SI #791H40	N/A	Replacement	No
A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	SI #796D05	N/A	Replacement	No
A193 Grade B7 Bolts	Unknown	Unknown	N/A	SI #791H40	N/A	Replacement	No
A194 Grade 2H Hex Nuts	Unknown	Unknown	N/A	SI #796D05	N/A	Replacement	No

7. Description of work: Replaced existing flange bolting on reactor head vent and reactor head spray flanges with new material. Existing material was identified as having excessive wear and some slight damage from previous disassembly and reassembly.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [ ] Not Applicable [X]

Test Pressure N/A psig Test Temperature N/A °F

9. Remarks: Bothe flanged connections were examined during system leakage test on 6/7/97, no leakage was identified.

Certificate of Compliance

We certify that the statements made in this report are correct and this REPLACEMENT Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 6-16 (Date), 1997

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the REPLACEMENT described in this report on 6-17, 19 97 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 6-17-97 Inspector: [signature] Commissions: IL932, NB7742NISB (State or Province, National Board)

CATEGORY 3

DAP 11-18
REVISION 07

FORM NIS-2 OWNER'S REPORT OF REPAIR OR REPLACEMENT
As Required by the Provisions of ASME Code Section XI

1. Owner: ComEd Company (Name) Date: 5-23-97
One First National Plaza, Chicago IL, 60690 (Address)
Sheet: 1 Of 1

2. Plant: Dresden Nuclear Power Station (Name)
6500 North Dresden Road, Morris IL., 60450 (Address) Unit: 3

3. Work Performed By: Same as Above (Name) WR 950065053 (PLAN 3-97-039)
Repair Organization P.O. No., Job No. etc.
Same as Above (Address)

4. Identification of System: 1100 Standby Liquid Control

5. (a) Construction Code USAS B31.1.0, 19 67 Edition, NO Addenda, Code Cases NONE
(b) Edition of Section XI used for Repair/Replacement 19 89 Edition, NO Addenda, Code Cases NONE

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Mfrs. Serial No.	Nat Brd No	Other ID	Yr Blt	Repair, Replaced or Replacement	Code Stamped Yes/No
3-1101-43B	Hancock	None	N/A	3-1101-43B	N/A	Repair	No

7. Description of work: Ground out existing bonnet seal weld to allow internal inspection. After inspection was completed, bonnet was reinstalled and seal welded per ComEd approved procedure.

8. Test Conducted: Hydrostatic [ ] Pneumatic [ ] Nominal Operating Pressure [X] Not Applicable [ ]

Test Pressure 1040 psig Test Temperature 73.2 °F

9. Remarks: VT-2 peformed during "B" Standby Liquid Contol injection test.

Certificate of Compliance

We certify that the statements made in this report are correct and this **REPAIR** Conforms to Section XI of the ASME Code.

Signed: Brendan J. Casey (Owner or Owner's Designee) ISI COORDINATOR (Title) 5-27 (Date), 19 97

Certificate of Inspection

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Illinois, employed by The Hartford Steam and Boiler Insurance and Inspection Co. of Hartford, Connecticut having inspected the **REPAIR** described in this report on 5-28, 1997 and state to the best of my knowledge and belief, this repair or replacement has been constructed in accordance with Section XI of the ASME Code. By signing this certificate neither the inspector nor his employer makes any warranty, expressed or implied, concerning the repair or replacement described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 5-28-97 Inspector: Kurt T Reimers Commissions: IL932, NB7742NISB
(State or Province, National Board)