

Commonwealth Edison Company
Dresden Generating Station
6500 North Dresden Road
Morris, IL 60450
Tel 815-942-2920



March 13, 1997

JSPLTR #97-0055

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

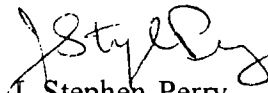
Enclosed is Licensee Event Report 97-005, Docket 50-237, which is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(b) which requires the reporting of any operation or condition prohibited by the plant's Technical Specifications.

This correspondence contains the following commitment:

1. Plant Engineering will monitor the low flow switch performance for the remainder of 1997 for trend evaluation. (NTS 237-180-97-00501)

If you have any questions, please contact Pete Holland, Dresden Regulatory Assurance Supervisor at (815) 942-2920 extension, 2714.

Sincerely,


J. Stephen Perry
Site Vice President
Dresden Station

Enclosure

cc: A. Bill Beach, Regional Administrator, Region III
NRC Resident Inspector's Office

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NRC FORM 366 (5-92)		U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95							
LICENSEE EVENT REPORT (LER)								ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.				
FACILITY NAME (1) Dresden Nuclear Power Station, Unit 2						DOCKET NUMBER (2) 05000237		PAGE (3) 1 of 4				
TITLE (4) High Pressure Coolant Injection Low Flow Setpoint Found Outside Technical Specification Limit Due to Setpoint Drift												
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER		
02	13	97	97	-- 005 --	00	03	13	97	FACILITY NAME	DOCKET NUMBER		
OPERATING MODE (9)		1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
POWER LEVEL (10)		085		20.2201(b)		20.2203(a)(3)(i)		50.73(a)(2)(iii)		73.71(b)		
				20.2203(a)(1)		20.2203(a)(3)(ii)		50.73(a)(2)(iv)		73.71(c)		
				20.2203(a)(2)(i)		20.2203(a)(4)		50.73(a)(2)(v)		OTHER		
				20.2203(a)(2)(ii)		50.36(c)(1)		50.73(a)(2)(vii)		(Specify in Abstract below and in Text, NRC Form 366A)		
				20.2203(a)(2)(iii)		50.36(c)(2)		50.73(a)(2)(viii)(A)				
				20.2203(a)(2)(iv)		X 50.73(a)(2)(i)		50.73(a)(2)(viii)(B)				
				20.2203(a)(2)(v)		50.73(a)(2)(ii)		50.73(a)(2)(x)				
LICENSEE CONTACT FOR THIS LER (12)												
NAME P. Garrett - Plant Engineering						Ext. 2713			TELEPHONE NUMBER (Include Area Code) (815) 942-2920			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)												
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		
X	BJ	FS	I204	N								
SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR		
YES (If yes, complete EXPECTED SUBMISSION DATE).					X	NO						

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On February 13, 1997, at 1530 with Unit 2 in mode 1(Run), while performing the scheduled quarterly Dresden Instrument Surveillance (DIS) 2300-02, High Pressure Coolant Injection Flow Calibration, the High Pressure Coolant Injection (HPCI) Low Flow Switch FS 2-2354, was identified with the trip setpoint below the Technical Specification limit. The cause for the Out-Of-Tolerance (OOT) is setpoint drift. The calibration frequency will be increased during the second quarter, 1997, to obtain additional trend data. The safety significance is minimal because the system would have operated as designed, providing 5600 gpm to the reactor vessel, and the pump's minimum flow requirements would have been satisfied during this system initiation.

NRC FORM 366A (5-92)		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95		
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FACILITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Dresden Nuclear Power Station, Unit 2		05000237	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 4
			97	-- 005 --	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - boiling water reactor - 2527 Mwt rated core thermal power.

Energy Industry Identification System (EIIS) codes are identified in the text as [XX] and are obtained from IEEE Standard 805-1984, IEEE Recommendation Practice for System Identification in Nuclear Power Plants and Related Facilities.

EVENT IDENTIFICATION:

High Pressure Coolant Injection Low Flow Setpoint Found Outside Technical Specification Limit Due To Setpoint Drift

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 2 Event Date: 02/13/97 Event Time: 1530 hrs
 Reactor Mode: 1 Mode Name: Run Power Level: 085
 Reactor Coolant System Pressure: 982 psig

B. DESCRIPTION OF EVENT:

This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(i)(b) which requires the reporting of any operation or condition prohibited by the plant's Technical Specifications.

On February 13, 1997, during the performance of High Pressure Coolant Injection [BJ] Flow surveillance DIS 2300-02, the low flow setpoint was found at 1.19 inches water column (inWC), outside of the Technical Specification (TS) limit of ≥ 600 gpm (equivalent to 2.82 inWC in accordance with instrument calculation NED-I-EIC-0109). Operations and Instrument Maintenance (IM) Supervision were notified. The switch was recalibrated to within specified tolerance.

A review of switch calibration data for February 13, 1997 indicated an anomaly (outside the allowable TS tolerance) from calibration data obtained during 1996 for flow switch FS 2-2354. The low flow switch setpoint was adjusted to within acceptable tolerance in accordance with DIS 2300-02, which includes a repeatability check of both the high flow and low flow switch following adjustment of either switch.

A review of calibration data did not reveal an adverse switch trend. Calibration frequency will be increased to perform two calibrations during the second quarter, 1997. The performance of the low flow switch will be monitored for any adverse trend. No other system or component inoperabilities have been identified which contribute to this event. In addition, no manual or automatic engineered safety feature (ESF) actuation occurred as a result of this event.

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C. CAUSE OF EVENT:

The root cause of this event is setpoint drift (NRC cause code X).

D. SAFETY ANALYSIS:

The impact of having the HPCI low flow switch setpoint set too low is the possibility of running the HPCI pump without the required minimum flow of 600 gallons per minute (GPM) when manually reducing flow following an automatic or manual initiation of HPCI. The ability of the system to perform its safety function during an accident would not have been affected. The system would have operated as designed, providing 5600 GPM to the reactor vessel, and the pump's minimum flow requirements would have been satisfied during this system initiation since the high flow switch was set as required. Additionally; the Isolation Condenser, Automatic Depressurization System, and low pressure systems (LPCI and Core Spray) were available during this event.

If the HPCI system were operated to control reactor vessel level or pressure in accordance with DOP 2300-03, High Pressure Coolant Injection System Manual Startup and Operation, flow to the reactor could have been reduced below the pump's minimum flow requirements. With the low flow switch actuation set low, the pump's minimum flow requirements would not have been satisfied during this condition and the potential would have existed for pump overheating and possible pump degradation.

During the time period that the low flow switch may have actuated low, the Unit 2 HPCI system was not operated in any mode that would have resulted in not meeting the pump's minimum flow requirements. The system was operated for a quarterly surveillance in accordance with DOS 2300-03, High Pressure Coolant System Operability Verification. During shutdown from the quarterly surveillance, DOS 2300-03, the HPCI pump momentarily transitions through low flow, but not of sufficient duration to cause pump degradation.

As a result, there was no impact on the health and safety of the public.

E. CORRECTIVE ACTIONS:

1. The surveillance performed on February 13, 1997, restored the switch setpoint to the appropriate setting. No further adjustments are necessary. (Complete)
2. Instrument Maintenance Department has scheduled two calibrations during the second quarter, 1997, to obtain additional trend data. (Complete)
3. Plant Engineering will monitor the low flow switch performance for the remainder of 1997 for trend evaluation. (NTS 237-180-97-00501)

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TEXT (If more space is required, use additional copies of NRC Form 366A) (*7)

F. PRIOR SIMILAR OCCURRENCES:

No previous occurrences were identified. The switch was recently added to the Technical Specification required surveillances as a result of the Technical Specification Upgrade, and no history of corrective actions exists.

G. COMPONENT FAILURE DATA:

Manufacturer	Nomenclature	Model Number
ITT Barton	Indicating Switch	289

An industry wide NPRDS data base search was performed on ITT Barton Model 289 indicating switches. Many events were identified which were associated with instrument setpoint drift.