

## UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 18, 1997

Ms. Irene Johnson, Acting Manager Nuclear Regulatory Services Commonwealth Edison Company Executive Towers West III 1400 Opus Place, Suite 500 Downers Grove, IL 60515

SUBJECT: REACTOR WATER CLEANUP SYSTEM HIGH ENERGY LINE BREAK OUTSIDE THE

DRYWELL (TAC NOS. M96442, M96443, M96444 AND M96445)

Dear Ms. Johnson:

By letters dated December 10, 1996, and January 14, 1997, Commonwealth Edison Company (ComEd) described the modification for the implementation of automatic isolation of the Reactor Water Cleanup System (RWCU) on direct indication of a high energy line break (HELB) for the Dresden Nuclear Power Station, Units 2 and 3, and the Quad Cities Nuclear Power Station, Units 1 and 2, as committed to by ComEd in a letter dated September 4, 1996. This letter also provided projected installation dates for the modifications at each of the sites.

Upon review of this submittal, the staff inquired of ComEd whether all resistance temperature devices (RTDs) to be used in this modification would be safety-related. In a telephone call on December 19, 1996, Mr. Mark Wagner of your staff confirmed that all RTDs for this modification will be safety-related at Dresden and Quad Cities.

We understand that the replacement of the intergranular stress-corrosion cracking (IGSCC) susceptible RWCU piping and components in Dresden, Unit 2, and Quad Cities, Units 1 and 2, have been replaced with IGSCC resistant materials and that the Dresden, Unit 3, replacement will be completed during its fourteenth refueling outage (currently scheduled for March 1997). The proposed schedule for installation of the auto isolation system in the fifteenth refueling outage for Dresden, Unit 2, and Quad Cities, Unit 1, is acceptable. However, due to the extended dates of the fifteenth refueling outage for Dresden, Unit 3 (01/99) and Quad Cities, Unit 2 (03/99) and the potential for further slippage in this schedule, it is requested that you reevaluate your commitment for these two units. Specifically, it is requested that you evaluate the feasibility of completing the non-outage work associated with these modifications on a reasonable schedule such that completion of the

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9702200321 970218 PDR ADDCK 05000237 PDR PDR modifications could be accomplished during a forced or scheduled outage of sufficient duration prior to each units respective fifteenth refueling outage.

Sincerely,

/s/

Robert M. Pulsifer, Project Manager Project Directorate III-2 Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Docket Nos. 50-237, 50-249, 50-254, 50-265

cc: see next page

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