October 28, 1996



U.S. Nuclear Regulatory Commission

Washington, DC 20555-0001

Attention:

Document Control Desk

Subject:

Braidwood Station Units 1 and 2 Byron Station Units 1 and 2 Dresden Station Units 2, and 3

LaSalle County Station Units 1 and 2 Quad Cities Station Units 1 and 2

Zion Station Units 1 and 2

Commonwealth Edison Company (ComEd) Response to

NRC Generic Letter (GL) 96-06, "ASSURANCE OF EQUIPMENT OPERABILITY AND CONTAINMENT INTEGRITY DURING DESIGN-BASIS ACCIDENT CONDITIONS," dated September 30, 1996.

NRC Dockets 50-454 and 50-455

NRC Dockets 50-456 and 50-457

NRC Dockets 50-237 and 50-249

NRC Dockets 50-373 and 50-374

NRC Dockets 50-254 and 50-265

NRC Dockets 50-295 and 50-304

Reference:

NRC Generic Letter 96-06, "ASSURANCE OF EQUIPMENT OPERABILITY AND

CONTAINMENT INTEGRITY DURING DESIGN-BASIS ACCIDENT

CONDITIONS", dated September 18, 1996.

The NRC staff issued the Referenced letter to (1) notify addresses about safety-significant issues that could affect containment integrity and equipment operability during accident conditions, (2) request that licensees submit certain information relative to the issues that have been identified and implement actions as appropriate to address these issues, and (3) require that all licensees submit a written response to the NRC relative to implementation of the requested actions.

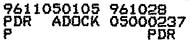
In GL 96-06 the NRC staff requested that all licensees implement the following actions:

Addressees are requested to determine:

- (1) if containment air cooler cooling water systems are susceptible to either waterhammer or two-phase flow conditions during postulated accident conditions;
- (2) if piping systems that penetrate the containment are susceptible to thermal expansion of fluid so that overpressurization of piping could occur.

In addition to the individual addressee's postulated accident conditions, these items should be reviewed with respect to the scenarios referenced in the generic letter.

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If systems are found to be susceptible to the conditions discussed in this generic letter, addresses are expected to assess the operability of affected systems and take corrective action as appropriate in accordance with the requirements stated in 10 CFR Part 50 Appendix B and as required by the plant Technical Specifications.

In GL 96-06, the NRC staff requested that all licensees provide a written response describing the following information:

- 1. Within 30 days from the date of this generic letter, addresses are required to submit a written response indicating: (1) whether of not the requested actions will be completed, (2) whether of not the requested information will be submitted and (3) whether of not the requested information will be submitted within the requested time period. Addressees who choose not to complete the requested actions, or choose not to submit the requested information, or are unable to satisfy the requested completion date, must describe in their response any alternative course of action that is proposed to be taken, including the basis for establishing the acceptability of the proposed alternative course of action and the basis for continued operability of affected systems and components as applicable.
- 2. Within 120 days of the date of this generic letter, addresses are requested to submit a written summary report stating actions taken in response to the requested actions noted above, conclusions that were reached relative to susceptibility for waterhammer and two-phase flow in the containment air cooler cooling water system and overpressurization of piping that penetrates containment, the basis for continued operability of affected systems and components as applicable, and corrective actions that were implemented or are planned to be implemented. If systems were found to be susceptible to the conditions that are discussed in this generic letter, identify the systems affected and describe the specific circumstances involved

ComEd has reviewed the requested information provided within Generic Letter 96-06. ComEd's 30-day response (Item 1 above) describing our implementation of the actions as described above for <u>each of our six stations</u> is summarized as follows:

ComEd has reviewed the Referenced Generic Letter and commits to implement the actions as described below for each of our six Nuclear Power Stations (Braidwood, Byron, Dresden, LaSalle County, Quad Cities and Zion Stations). Susceptibility of the air cooling water systems to the conditions arising during post accident mitigation will be reviewed. This review will address the combined LOCA (Loss of Coolant Accident) and LOOP (Loss of Offsite Power) described in Information Notice 96-45, "Potential common mode Post-Accident Failure of Containment Coolers." Evaluation of thermal expansion and potential susceptibility of piping systems inside containment or drywell that will see increased temperature during accident mitigation will also be addressed.

ComEd is an active participant in the Westinghouse Owners Group and BWR Owners Group. Any appropriate lessons learned identified as a result of our participation in these owners groups will be incorporated into our programs, where applicable.

ComEd's 120-day response will be provided to the NRC staff prior to January 28, 1997, in accordance with the requested schedule listed in GL 96-06.

To the best of my knowledge and belief, the information contained herein is true and accurate.

Please direct any questions concerning this response to this office.

Sincerely,

John B. Hosmer

Vice President

Signed before me on this __

_ day,

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cc:

- A. Beach, Regional Administrator-RIII
- R. Capra, Director of Directorate III-2, NRR
- G. Dick, Byron Project Manager, NRR
- R. Assa, Braidwood Project Manager, NRR
- J. Stang, Dresden Project Manager, NRR
- D. Skay, LaSalle Project Manager, NRR
- R. Pulsifer, Quad Cities Project Manager, NRR
- C. Shiraki, Zion Project Manager, NRR
- C. Phillips, Senior Resident Inspector (Braidwood)
- S. Burgess, Senior Resident Inspector (Byron)
- C. Vanderniet, Senior Resident Inspector (Dresden)
- M. Huber, Senior Resident Inspector (LaSalle)
- C. Miller, Senior Resident Inspector (Quad Cities)
- R. Westberg, Acting Senior Resident Inspector (Zion)

Office of Nuclear Facility Safety - IDNS