October 8, 1996

Ms. Irene Johnson, Acting Manager **Nuclear Regulatory Services** Commonwealth Edison Company Executive Towers West III 1400 Opus Place, Suite 500 Downers Grove, IL 60515

SUBJECT:

EVALUATION OF INSERVICE TESTING RELIEF REQUEST - DRESDEN NUCLEAR

POWER STATION, UNIT 3 (TAC NO. M96507)

Dear Ms. Johnson:

By letter dated September 5, 1996, you submitted a one-time request for relief from certain provisions of Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) pursuant to 10 CFR 50.55a for certain plant safety/relief valves.

As required by 10 CFR 50.55a, inservice testing (IST) of certain ASME Code Class 1, 2, and 3 pumps and valves must be performed in accordance with Section XI of the ASME Code, except where alternatives are authorized or relief is granted by the Commission pursuant to paragraphs (a)(3)(i), (a)(3)(ii), or (f)(6)(i). In order to obtain authorization or relief, the licensee must demonstrate that (1) the proposed alternatives provide an acceptable level of quality and safety, (2) compliance would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety, or (3) conformance is impractical for its facility. Guidance on acceptable alternatives to Section XI requirements for certain aspects of the IST program was provided by the NRC in Generic Letter 89-04.

The staff has reviewed the relief request proposed in your September 5, 1996, letter. The alternative you have proposed to the ASME Code testing of the main steam safety valves and the safety/relief valve is authorized pursuant to 10 CFR 50.55a(a)(3)(ii) in that compliance with the code requirements would result in hardship without a compensating increase in the level of quality and safety. The specific details are contained in the enclosed SE.

Sincerely,

Original signed by: Clyde Shiraki for

Robert A. Capra, Project Director Project Directorate III-2 Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

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Docket No. 50-249

Enclosure: Safety Evaluation

cc w/encl: see next page

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PUBLIC R. Capra

PDIII-2 r/f ACRS, T2E26 J. Stang C. Moore

J. Roe, JWR OGC, 015B19

P. Hiland, RIII

G. Hill (4), T5C3

G. Hammer, 07E23

*SEE PREVIOUS CONCURRENCE. DOCUMENT NAME: DR96507.LTR

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DATE	10/96	10/8/96	10/7/96	10/04/96	10/8/96



UNITED STATES CLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

Robert A. Capra, Project Director

Project Directorate III-2

Division of Reactor Projects - III/IV Office of Nuclear Reactor Regulation

Docket No. 50-249

Enclosure: Safety Evaluation

cc w/encl: see next page

I. Johnson Commonwealth Edison Company Dresden Nuclear Power Station Unit Nos. 2 and 3

cc:

Michael I. Miller, Esquire Sidley and Austin One First National Plaza Chicago, Illinois 60603

Site Vice President Dresden Nuclear Power Station 6500 North Dresden Road Morris, Illinois 60450-9765

Station Manager Dresden Nuclear Power Station 6500 North Dresden Road Morris, Illinois 60450-9765

U.S. Nuclear Regulatory Commission Resident Inspectors Office Dresden Station 6500 North Dresden Road Morris, Illinois 60450-9766

Regional Administrator U.S. NRC, Region III 801 Warrenville Road Lisle, Illinois 60532-4351

Illinois Department of Nuclear Safety Office of Nuclear Facility Safety 1035 Outer Park Drive Springfield, Illinois 62704

Chairman Grundy County Board Administration Building 1320 Union Street Morris, Illinois 60450

Document Control Desk-Licensing Commonwealth Edison Company 1400 Opus Place, Suite 400 Downers Grove, Illinois 60515