

Dominion Nuclear Connecticut, Inc.
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JUN 27 2017

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No. 17-271
NSS&L/MLC R0
Docket No. 50-336
License No. DPR-65

DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION UNIT 2
INSERVICE INSPECTION PROGRAM – OWNER’S ACTIVITY REPORT, REFUELING
OUTAGE 24

Dominion Nuclear Connecticut, Inc. hereby submits the American Society of Mechanical Engineers (ASME), Section XI, Form OAR-1, Owner’s Activity Report for the period from November 8, 2015 through Refueling Outage 24, completed on May 5, 2017, for Millstone Power Station Unit 2. The enclosure is in accordance with the requirements of ASME Code Case N-532-4.

If you have any questions or require additional information, please contact Jeff Langan at (860) 444-5544.

Sincerely,

B. L. Stanley
Director, Nuclear Station Safety and Licensing

Enclosure:

Millstone Power Station Unit 2, Owner’s Activity Report - Refueling Outage 24

Commitments made in this letter:

None

AD47
NRR

cc: U.S. Nuclear Regulatory Commission
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NRC Senior Resident Inspector
Millstone Power Station

Serial No. 17-271
Docket No. 50-336

ENCLOSURE

INSERVICE INSPECTION PROGRAM – OWNER'S ACTIVITY REPORT

REFUELING OUTAGE 24

**MILLSTONE POWER STATION UNIT 2
DOMINION NUCLEAR CONNECTICUT, INC.**

MILLSTONE POWER STATION

UNIT NO. 2

OWNER'S ACTIVITY REPORT

REFUELING OUTAGE 24

Revision 0

Contents:

OAR-1 Report Number: MP2-2R24

Table 1: Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service.

Table 2: Abstract of Repairs/Replacement Activities Required for Continued Service

Prepared By:

R. Zieber
ISI Program Owner

Date:

06/08/2017

Reviewed By:

H. Shelton
Independent Review

Date:

06/12/2017

Reviewed By:

Z. Adesh
Authorized Nuclear Inservice Inspector

Date:

06/13/2017



Form OAR-1 Owner's Activity Report

Attachment 1, ER-AA-ISI-100

Page 1 of 6

Report Number: MP2-2R24

Plant Millstone Nuclear Power Station, Rope Ferry Road, Waterford, Connecticut 06385

Unit No. 2 Commercial service date 12/26/1975 Refueling outage no. 24
(if applicable)

Current inspection interval 4th
(1st, 2nd, 3rd, 4th, other)

Current inspection period 3rd
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plans 2004 Edition, No Addenda

Date and revision of inspection plans 03/20/17 Revision 3, Change 4-004

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans
N/A

Code Cases used: N-460, N-532-4, N-566-2, N-722-1, N-729-1, N-770-1
(if applicable)

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of 2R24 conform to the requirements of Section XI.
(refueling outage number)

Signed *Michael A. Zick* ISI Program Owner Date 06/08/2017
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Connecticut and employed by The Hartford Steam Boiler Inspection and Insurance Company of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Elizabeth York Commissions *AKC AKI*
Inspector's Signature *NS 9584 CT NSI INCR*
National Board, State, Province and Endorsements

Date *June 13, 2017*



Table 1 Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
F-A / F1.30B	Pipe Support 413122	Bolting thread engagement evaluated by Engineering as acceptable for continued service as documented in UIR MP2-17-001, CR1061880
F-A / F1.20	Pipe Support 410077	Bolting thread engagement evaluated by Engineering as acceptable for continued service as documented in UIR MP2-17-002, CR1062470
F-A / F1.20C	Pipe Support 302027	Bolting thread engagement evaluated by Engineering as acceptable for continued service as documented in UIR MP2-17-003, CR1063534
F-A / F1.30G	Pipe Support 413153	Bolting thread engagement evaluated by Engineering as acceptable for continued service as documented in UIR MP2-17-004, CR1064227
F-A / F1.10E	Pipe Support 410032	Bolting thread engagement evaluated by Engineering as acceptable for continued service as documented in UIR MP2-17-005, CR1064652
F-A / F1.10B	Pipe Support 510016-A	Bolting thread engagement evaluated by Engineering as acceptable for continued service as documented in UIR MP2-17-006, CR1064730
F-A / F1.10H	Pipe support 402113-L	Missing washer from snubber support end paddle evaluated by Engineering as acceptable for continued service as documented in UIR-MP2-17-008, CR1065024. Corrective action scheduled for washer replacement to restore support to design condition.
F-A / F1.10C	Pipe Support 310049	Support rod in contact with conduit evaluated by Engineering as acceptable for continued service as documented in UIR-MP2-17-009, CR1065325. Corrective action scheduled for support rod adjustment to restore support to design condition. Support load setting evaluated by Engineering as acceptable for continued service as documented in UIR-MP2-17-013, CR1065491
F-A / F1.20E	Pipe Support 412010	Bolting thread engagement evaluated by Engineering as acceptable for continued service as documented in UIR MP2-17-014, CR1065914



Table 1 Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
C-H / C7.10	Valve 2-CH-037	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2015-1189 and ETE MP-2017-1036
C-H / C7.10	Valve 2-CH-184	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2015-1189
C-H / C7.10	Valve 2-CH-193	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2015-1189
C-H / C7.10	Valve 2-SI-470	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1045
C-H / C7.10	Pump P41B suction elbow flange	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1043
C-H / C7.10	Pump P41B casing to stuffing box cover	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1019
C-H / C7.10	Valve 2-CH-514	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1036
C-H / C7.10	Pump P43A casing to stuffing box cover	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1028
C-H / C7.10	Valve 2-CS-2B	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1013



Table 1 Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
C-H / C7.10	Pump P43B casing to stuffing box cover	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1012
C-H / C7.10	Valve 2-SI-625	Evidence of leakage at bolted connection evaluated in accordance with ASME Code Case N-566-2 as acceptable for continued service as documented in ETE MP-2017-1046
C-H / C7.10	Pump P19B Stuffing box cover	Through wall leakage identified in pump stuffing box cover as documented in CR1061230 and CR1061348. Alternative Request RR-04-26 (Dominion letter S/N 17-187), dated 4/28/17, Accession No ML17123A043, submitted to NRC to request approval of a proposed alternative to keep the pump available for service with the identified indications until a replacement part can be procured and a permanent code repair can be made. NRC approval granted by letter dated 5/3/17, Accession No. ML17122A374.
Code Case N-722-1 / B15.205	Loop 2B Cold Leg Instrument Tap M2TE-122CD	Visual examination identified staining on cold leg instrument tap nozzle. Condition evaluated by Engineering as acceptable for continued service. Staining was determined not to be the result of reactor coolant leakage. Reference CR 1064672



Table 1 Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service

Examination Category and Item Number	Item Description	Evaluation Description
Code Case N- 729-1/ B4.30	Reactor Vessel Head Penetrations	Visual examination identified area of white residue located at CEDM housing numbers 26, 38 and 42. Condition evaluated by Engineering as acceptable for continued services. Residue was determined to not be the result to reactor head penetration leakage. Residue at CEDM penetration 26 and 42 were found to be caused by leakage from a horizontal piping flange above, that when disassembled during the outage, overflow had dripped down to the area on the vessel head. Residue on CEDM housing 38 was analyzed and found to be foreign material that did not contain reactor coolant. The subject areas have been cleaned and visually inspected with no material degradation identified. Reference. CRs 1065275, 1065281 and 1065498.



Table 2 Abstract of Repair/Replacement Activities Required for Continued Service

Code Class	Item Description	Description of Work	Date Completed	Repair/Replacement Plan Number
1	Mechanical Snubber	Replace damaged thrust bearing assembly	11/30/2015	53102483719
1	Mechanical Snubber	Replace degraded bearing retaining nut	01/04/2016	53102579825
2	Pipe Spool	Removal of surface indications on Safety Injection line 6-GCB-10	03/17/2016	53102831249
3	Pipe Spool	Replace Service Water Pipe Spool SK-905A	04/25/2017	53103064542