Facility:	Hope C	reek	2016	5 NR	С	Dat	e of l	Exan	n:		08/	/22/1	6					
					RO ŀ	(/A (Categ	ory F	oints	5				SR	0-0	nly P	oints	
Tier	Group	K 1	К 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	А	2	G	*	Total
1.	1	4	3	3				3	4			3	20	3	3	4	1	7
Emergency &	2	1	1	2				1	1			1	7	2	2		1	3
Plant Evolutions	Tier Totals	5	4	5				4	5			4	27	Ę	5	Ę	5	10
2	1	2	2	2	3	2	3	2	2	3	2	3	26	2	2		3	5
2. Plant	2	1	1	1	1	2	1	1	1	1	1	1	12	0	2		1	3
Plant Systems	Tier Totals	3	3	3	4	4	4	3	3	4	3	4	38	2	ł	2	4	8
3. Generic K	ies		1	2	2	-	3	2	1	10	1	2	3	4	7			
(Categories	5				3		2		3		2	10	2	2	1	2	/

Note: 1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).

- 2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.
- 3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to section D.1.b of ES-401, for guidance regarding elimination of inappropriate K/A statements.
- 4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.
- 5. Absent a plant specific priority, only those KAs having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
- 6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
- 7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/A's
- 8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IR) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above. If fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.
- 9. For Tier 3, select topics from Section 2 of the K/A Catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-40I-3. Limit SRO selections to K/As that are linked to 10CFR55.43

Hope Creek 2016 NRC Written Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1 Group 1

EAPE # / Name Safety Function	К1	К2	К3	A1	A2	G	K/A Topic(s)	lmp.	Q#
295024 High Drywell Pressure / 5					×		EA2.02 - Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE: Drywell temperature	4,0	76
295016 Control Room Abandonment / 7					x		AA2.05 - Ability to determine and/or interpret the following as they apply to CONTROL ROOM ABANDONMENT : Drywell pressure	3.9	77
295026 Suppression Pool High Water Temp, / 5					x		EA2.02 - Ability to determine and/or interpret the following as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE: Suppression pool level	3.9	78
295006 SCRAM / 1						×	 2.2.39 - Equipment Control: Knowledge of less than one hour technical specification action statements for systems. 	4.5	79
295038 High Off-site Release Rate / 9						x	2.4.41 - Emergency Procedures / Plan: Knowledge of the emergency action level thresholds and classifications.	4.6	80
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4						X	2.4.21 - Emergency Procedures / Plan: Knowledge of the parameters and logic used to assess the status of safety functions, such as reactivity control, core cooling and heat removal, reactor coolant system integrity, containment conditions, radioactivity release control, etc.	4.6	81
295004 Partial or Total Loss of DC Pwr / 6						x	2.2.40 - Equipment Control: Ability to apply technical specifications for a system.	4.7	82
295006 SCRAM / 1	x						AK1.02 - Knowledge of the operational implications of the following concepts as they apply to SCRAM : Shutdown margin	3.4	39
295031 Reactor Low Water Level / 2	x						EK1.03 - Knowledge of the operational implications of the following concepts as they apply to REACTOR LOW WATER LEVEL : Water level effects on reactor power	3.7	40
600000 Plant Fire On-site / 8	x					Jana and	AK1.02 - Knowledge of the operation applications of the following concepts as they apply to Plant Fire On Site: Fire Fighting	2.9	41
295005 Main Turbine Generator Trip / 3		x					AK2.03 - Knowledge of the interrelations between MAIN TURBINE GENERATOR TRIP and the following: Recirculation system	3.2	42
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4		x					AK2.06 - Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION and the following: Reactor power	3.8	43
295019 Partial or Total Loss of Inst. Air / 8		x					AK2.14 - Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF INSTRUMENT AIR and the following: Plant air systems	3.2	44
295025 High Reactor Pressure / 3			x				EK3.02 - Knowledge of the reasons for the following responses as they apply to HIGH REACTOR PRESSURE : Recirculation pump trip: Plant-Specific	3.9	45
295026 Suppression Pool High Water Temp. / 5			x				EK3.02 - Knowledge of the reasons for the following responses as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE: Suppression pool	3.9	46

295037 SCRAM Conditions Present and Reactor Power Above

K/A Category Totals:

APRM Downscale or Unknown / 1

Х

4

3

3

3

4/3

3/4

Hope Creek 2016 NRC Written Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1 Group 1

EAPE # / Name Safety Function	K1	К2	К3	A1	A2	G	K/A Topic(s)	Imp.	Q#
	T	· · · · ·							
					A Contraction of the	- House I	cooling		
295016 Control Room Abandonment / 7			x				AK3.01 - Knowledge of the reasons for the following responses as they apply to CONTROL ROOM ABANDONMENT : Reactor SCRAM	4.1	47
295004 Partial or Total Loss of DC Pwr / 6				x			AA1.01 - Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER : D.C. electrical distribution systems	3.3	48
295024 High Drywell Pressure / 5				x			EA1.17 - Ability to operate and/or monitor the following as they apply to HIGH DRYWELL PRESSURE: Containment spray: Plant-Specific	3.9	49
295023 Refueling Acc Cooling Mode / 8				x			AA1.02 - Ability to operate and/or monitor the following as they apply to REFUELING ACCIDENTS : Fuel pool cooling and cleanup system	2.9	50
295028 High Drywell Temperature / 5					×		EA2.06 - Ability to determine and/or interpret the following as they apply to HIGH DRYWELL TEMPERATURE : Torus/suppression chamber air space temperature: Plant-Specific	3.4	51
295038 High Off-site Release Rate / 9					x		EA2.03 - Ability to determine and/or interpret the following as they apply to HIGH OFF-SITE RELEASE RATE : Radiation levels	3.5	52
295003 Partial or Complete Loss of AC / 6					X		AA2.04 - Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER : System lineups	3.5	53
295018 Partial or Total Loss of CCW / 8						x	2.4.8 - Emergency Procedures / Plan: Knowledge of how abnormal operating procedures are used in conjunction with EOP's.	3.8	54
700000 Generator Voltage and Electric Grid Disturbances						X	2.1.27 - Conduct of Operations: Knowledge of system purpose and / or function.	3.9	55
295021 Loss of Shutdown Cooling / 4						x	2.1.19 - Conduct of Operations: Ability to use plant computers to evaluate system or component status.	3.9	56
295030 Low Suppression Pool Water Level / 5					×		EA2.04 - Ability to determine and/or interpret the following as they apply to LOW SUPPRESSION POOL WATER LEVEL : Drywell/ suppression chamber differential pressure: Mark-I&II	3.5	57

EK1.03 - Knowledge of the operational implications of the following concepts as they apply to SCRAM CONDITION

PRESENT AND REACTOR POWER

Group Point Total:

ABOVE APRM DOWNSCALE OR UNKNOWN: Boron effects on reactor

power (SBLC)

4.2

58

20/7

Hope Creek 2016 NRC Written Examination Outline Emergency and Abnormal Plant Evolutions – Tier 1 Group 2

					-				
EAPE # / Name Safety Function	K1	K2	К3	A1	A2	G	K/A Topic(s)	lmp.	Q#

									-
295017 High Off-site Release Rate / 9					x		AA2.05 - Ability to determine and/or interpret the following as they apply to HIGH OFF-SITE RELEASE RATE : Meteorological data	3.8	83
295015 Incomplete SCRAM / 1						x	2.1.32 - Conduct of Operations: Ability to explain and apply all system limits and precautions.	4.0	84
295008 High Reactor Water Level / 2					X .		AA2.04 - Ability to determine and/or interpret the following as they apply to HIGH REACTOR WATER LEVEL : Heatup rate: Plant-Specific	3.3	85
500000 High CTMT Hydrogen Conc. / 5	x						EK1.01 - Knowledge of the operational implications of the following concepts as they apply to HIGH CONTAINMENT HYDROGEN CONCENTRATIONS: Containment integrity	3. 3	59
295010 High Drywell Pressure / 5		x					AK2.04 - Knowledge of the interrelations between HIGH DRYWELL PRESSURE and the following: Nitrogen makeup system: Plant-Specific	2.6	60
295014 Inadvertent Reactivity Addition / 1			x				AK3.02 - Knowledge of the reasons for the following responses as they apply to INADVERTENT REACTIVITY ADDITION: Control rod blocks	3. 7	61
295015 Incomplete SCRAM / I				х		la ^{liter}	AA1.02 - Ability to operate and/or monitor the following as they apply to INCOMPLETE SCRAM : RPS	4. 0	62
295009 Low Reactor Water Level / 2					x		AA2.01 - Ability to determine and/or interpret the following as they apply to LOW REACTOR WATER LEVEL : Reactor water level	4. 2	63
295008 High Reactor Water Level / 2						×	2.4.49 - Emergency Procedures / Plan: Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	4. 6	64
295012 High Drywell Temperature / 5			x				AK3.01 - Knowledge of the reasons for the following responses as they apply to HIGH DRYWELL TEMPERATURE : Increased drywell cooling	3. 5	65
K/A Category Totals:	1	1	2	1	1/2	1/1	Group Point Total:		7 /3

٦

Hope Creek 2016 NRC Written Examination Outline Plant Systems – Tier 2 Group 1

System # / Name	К 1	К 2	К 3	к 4	К 5	К 6	A 1	A2	A 3	A 4	G		Imp	Q#
			_											
262002 UPS (AC/DC)								*				A2.03 - Ability to (a) predict the impacts of the following on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Frequency changes in the system	2.6	86
264000 EDGs								×				A2.07 - Ability to (a) predict the impacts of the following on the EMERGENCY GENERATORS (DIESEL/JET); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss of off-site power during full-load testing	3.7	87
263000 DC Electrical Distribution											×	2.4.49 - Emergency Procedures / Plan: Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	4.4	88
212000 RPS											×	2.2.22 - Equipment Control: Knowledge of limiting conditions for operations and safety limits.	4.7	89
215003 IRM											x	2.4.2 - Emergency Procedures / Plan: Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.	4.6	90
262001 AC Electrical Distribution	×											K1.01 - Knowledge of the physical connections and/or cause- effect relationships between A.C. ELECTRICAL DISTRIBUTION and the following: Emergency generators (diesel/jet)	3.8	1
206000 HPCI	x											K1.08 - Knowledge of the physical connections and/or cause- effect relationships between HIGH PRESSURE COOLANT INJECTION SYSTEM and the following: A.C. power: BWR-2,3,4	3.0	2
218000 ADS		x										K2.01 - Knowledge of electrical power supplies to the following: ADS logic	3.1	3
205000 Shutdown Cooling		x										K2.01 - Knowledge of electrical power supplies to the following: Pump motors	3.1	4
212000 RPS			x									K3.06 - Knowledge of the effect that a loss or malfunction of the REACTOR PROTECTION SYSTEM will have on following: Scram air header solenoid operated valves	4.0	5

Hope Creek 2016 NRC Written Examination Outline Plant Systems – Tier 2 Group 1

System # / Name	к 1	К 2	К 3	K 4	K 5	K 6	A 1	A2	A 3	A 4	G		Imp	Q#
	1	T		1	T	Γ	1			Γ		K3.02 - Knowledge of the effect		
300000 Instrument Air			x									that a loss or malfunction of the (INSTRUMENT AIR SYSTEM) will have on the following: Systems having pneumatic valves and controls	3.3	6
264000 EDGs				x								K4.07 - Knowledge of EMERGENCY GENERATORS (DIESEL/JET) design feature(s) and/or interlocks which provide for the following: Local operation and control	3.3	7
261000 SGTS				×	K4.01 - Knowledge of STANDBY GAS TREATMENT SYSTEM design feature(s) and/or interlocks which provide for the following: Automatic system initiation K5.02 - Knowledge of the		3.7	8						
211000 SLC				X X System Initiation K5.02 - Knowledge of the operational implications of the following concepts as they apply to STANDBY LIQUID CONTROL SYSTEM : Chugging (as it pertains to boron mixing) K5.05 - Knowledge of the		2.8	9							
239002 SRVs					×							(as it pertains to boron mixing) K5.05 - Knowledge of the operational implications of the following concepts as they apply to RELIEF/SAFETY VALVES : Discharge line quencher operation		10
209001 LPCS						x						K6.08 - Knowledge of the effect that a loss or malfunction of the following will have on the LOW PRESSURE CORE SPRAY SYSTEM : Keep fill system	2.9	11
203000 RHR/LPCI: Injection Mode						x						K6.12 - Knowledge of the effect that a loss or malfunction of the following will have on the RHR/LPCI: INJECTION MODE (PLANT SPECIFIC) : ECCS room integrity	2.7	12
215004 Source Range Monitor							x					A1.04 - Ability to predict and/or monitor changes in parameters associated with operating the SOURCE RANGE MONITOR (SRM) SYSTEM controls including: Control rod block status		13
263000 DC Electrical Distribution							x					A1.01 - Ability to predict and/or monitor changes in parameters associated with operating the D.C. ELECTRICAL DISTRIBUTION controls including: Battery charging/discharging rate	2.5	14

Hope Creek 2016 NRC Written Examination Outline Plant Systems – Tier 2 Group 1

System # / Name	К 1	к 2	К 3	к 4	к 5	к 6	A 1	A2	A 3	A 4	G		Imp	Q#
223002 PCIS/Nuclear Steam Supply Shutoff								X				A2.08 - Ability to (a) predict the impacts of the following on the PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Surveillance testing	2.7	15
262002 UPS (AC/DC)								×				A2.01 - Ability to (a) predict the impacts of the following on the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.); and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Under voltage	2.6	16
215003 IRM									×			A3.01 - Ability to monitor automatic operations of the INTERMEDIATE RANGE MONITOR (IRM) SYSTEM including: Meters and recorders	3.3	17
215005 APRM / LPRM									×			A3.08 - Ability to monitor automatic operations of the AVERAGE POWER RANGE MONITOR/LOCAL POWER RANGE MONITOR SYSTEM including: Control rod block status	3.7	18
259002 Reactor Water Level Control										×		A4.02 - Ability to manually operate and/or monitor in the control room: All individual component controllers in the automatic mode	3.7	19
217000 RCIC										x		A4.01 - Ability to manually operate and/or monitor in the control room: RCIC turbine speed	3.7	20
400000 Component Cooling Water												2.4.30 - Emergency Procedures / Plan; Knowledge of events related to system operation / status that must be reported to internal organizations or external agencies, such as the state, the NRC, or the transmission system operator.	2.7	21
206000 HPC1											×	2.2.22 - Equipment Control: Knowledge of limiting conditions for operations and safety limits.	4.0	22
300000 Instrument Air				×								K4.02 - Knowledge of (INSTRUMENT AIR SYSTEM) design feature(s) and or interlocks which provide for the following: Cross-over to other air systems	3.0	23

Hope Creek 2016 NRC Written Examination Outline Plant Systems – Tier 2 Group 1

System # / Name	К 1	К 2	К 3	K 4	K 5	К 6	A 1	A2	A 3	A 4	G		Imp	Q#
400000 Component Cooling Water									x			A3.01 - Ability to monitor automatic operations of the CCWS including: Setpoints on instrument signal levels for normal operations,warnings, and trips that are applicable to the CCWS	3.0	24
217000 RCIC						x						K6.01 - Knowledge of the effect that a loss or malfunction of the following will have on the REACTOR CORE ISOLATION COOLING SYSTEM (RCIC): Electrical power	3.4	25
211000 SLC											×	2.4.20 - Emergency Procedures / Plan: Knowledge of operational implications of EOP warnings, cautions, and notes.		26
K/A Category Totals:	2	2	2	3	2	3	2	2 /2	3	2	3/3	Group Point Total:	2	6/5

Hope Creek 2016 NRC Written Examination Outline Plant Systems – Tier 2 Group 2

System # / Name	к 1	к 2	К 3	к 4	к 5	К 6	A 1	A2	A 3	A 4	G		Imp.	Q #
	<u>A</u>	<u></u>	<u>h</u>	<u> </u>		<u></u>	<u> </u>			<u> </u>				
290003 Control Room HVAC								X				A2.04 - Ability to (a) predict the impacts of the following on the CONTROL ROOM HVAC; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Initiation/failure of fire protection system	3.3	91
245000 Main Turbine Gen. / Aux.											X	2.4.46 - Emergency Procedures / Plan: Ability to verify that the alarms are consistent with the plant conditions.	4.2	92
215002 RBM	RBM X A2.02 - Ability to (a) predict the impacts of the following on the ROD BLOCK MONITOR SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss or reduction in recirculation system flow (flow comparator): BWR-3,4.5 K1.01 - Knowledge of the physical connections and/or cause- effect		3.3	93										
290001 Secondary CTMT	20001 Secondary CTMT X										K1.01 - Knowledge of the physical connections and/or cause- effect relationships between SECONDARY CONTAINMENT and the following: Reactor building ventilation: Plant-Specific	3.3	27	
226001 RHR/LPCI: CTMT Spray Mode		x										K2.02 - Knowledge of electrical power supplies to the following: Pumps	2.9	28
223001 Primary CTMT and Aux.	CHR/LPCT: CTMT X power supplies to the following: Pumps Primary CTMT and X K3.08 - Knowledge of the effect that a loss or malfunction of the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES will have on following: Pneumatically operated valves internal to containment/drywell: Dure Greefer		2.7	29										
201001 CRD Hydraulic				x								K4.05 - Knowledge of CONTROL ROD DRIVE HYDRAULIC SYSTEM design feature(s) and/or interlocks which provide for the following: Control rod SCRAM	3.8	30
271000 Off-gas					x							K5.06 - Knowledge of the operational implications of the following concepts as they apply to OFFGAS SYSTEM : Catalytic recombination	2.7	31
290003 Control Room HVAC						x					hip sub-	K6.01 - Knowledge of the effect that a loss or malfunction of the following will have on the CONTROL ROOM HVAC : Electrical power	2.7	32

Hope Creek 2016 NRC Written Examination Outline Plant Systems – Tier 2 Group 2

System # / Name	К 1	к 2	к 3	К 4	к 5	К 6	A 1	A2	A 3	A 4	G		Imp.	Q #
216000 Nuclear Boiler Inst.							x					A1.04 - Ability to predict and/or monitor changes in parameters associated with operating the NUCLEAR BOILER INSTRUMENTATION controls including: System venting	2.6	33
272000 Radiation Monitoring								X				A2.16 - Ability to predict the impacts of the following on the RADIATION MONITORING SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Instrument malfunctions	2.7	34
241000 Reactor/Turbine Pressure Regulator									×			A3.10 - Ability to monitor automatic operations of the REACTOR/TURBINE PRESSURE REGULATING SYSTEM including: Main stop/throttle valve operation	3.3	35
215001 Traversing In-core Probe										x		A4.03 - Ability to manually operate and/or monitor in the control room: Isolation valves: Mark-I&II(Not-BWR1)	3.0	36
259001 Reactor Feedwater											×	2.2.36 - Equipment Control: Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations.	3.1	37
233000 Fuel Pool Cooling/Cleanup		-			x							K5.07 - Knowledge of the operational implications of the following concepts as they apply to FUEL POOL COOLING AND CLEAN-UP : Maximum (abnormal) heat 102d	2.5	38
K/A Category Totals:	1	1	1	1	2	1	1	1/2	1	1	1/1	Group Point Total:		12 /3

Facility:	Hope Cr	eek 2016 NRC Date: 08/22/1	6			
Cotogony	V/A #	Tapia	R	0	SRO	-Only
Category	N/A#	Торіс	IR	Q#	IR	Q#
	2.1.41	Knowledge of the refueling process.			3.7	94
	2.1.6	Ability to manage the control room crew during plant transients.			4.8	100
1. Conduct	2.1.29	Knowledge of how to conduct system lineups, such as valves, breakers, switches, etc.	4.1	66		
	2.1.34	Knowledge of primary and secondary plant chemistry limits.	2.7	67		
	2.1.27	Knowledge of system purpose and / or function.	3.9	75		
	Subtotal			3		2
	2.2.44	Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives effect plant and system conditions.			4.4	95
	2.2.40	Ability to apply technical specifications for a system.			4.7	98
2.						
Equipment						
Control	2.2.39	Knowledge of less than one hour technical specification action statements for systems.	3.9	68		
	2.2.14		3.9	69		
	Subtotal			2		2
	2.3.15	Knowledge of radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personnel monitoring equipment, etc.			3.1	96
3. Radiation			_			
Radiation Control	2.3.5	Ability to use radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personell monitoring equipment, etc.	2.9	70		
	2.3.15	Knowledge of radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personnel monitoring equipment, etc.	2.9	71		

	2.3.13	Knowledge of Radialogical Safety Procedures pertaining to licensed operator duties, such as response to radiation monitor alarms, containment entry requirements, fuel handling responsibilities, access to locked high radiation areas, aligning filters, etc.	3.4	74		
	Subtotal			3		1
	2.4. 1 1	Knowledge of abnormal condition procedures.			4.2	97
	2.4.22	Knowledge of the bases for prioritizing safety functions during abnormal/emergency operations.			4.4	99
4.						
Emergency						
Procedures /	2.4.6	Knowledge of EOP mitigation strategies.	3.7	72		
Fidii	2.4.28	Knowledge of procedures relating to a security event.	3.2	73		
	Subtotal			2		2
Tier 3 Point Tota		10		7		

Record of Rejected K/As

Tier / Group	Randomly Selected K/A	Reason for Rejection
2/1	K5.02	#9: Cannot write an operationally valid question. Randomly selected K5.04
2/1	A2.08	#15: Cannot write an operationally valid question. Randomly selected A2.03
2/1	A2.01	#16: Cannot write an operationally valid question. Randomly selected K4.01
2/1	2.4.30	#21: Cannot write an operationally valid question. Randomly selected 2.4.11
2/1	2.2.22	#22: Cannot write an RO level of knowledge question on this K/A. Randomly selected 2.2.39
2/2	K2.02	#28: K/A subject is oversampled (same as question #4). Randomly selected A3.05
2/2	K5.06	#31: Cannot write an operationally valid question. Randomly selected A1.08
2/2	A1.04	#33: Cannot write an operationally valid question. Randomly selected A1.01
2/2	2.2.36	#37: Cannot write an operationally valid question. Randomly selected 2.2.44
2/2	K5.07	#38: Cannot write an operationally valid question. Randomly selected K5.06
1/1	EA2.06	#51: Cannot write an operationally valid question. Randomly selected EA2.03
1/1	2.4.8	#54: Cannot write an operationally valid question. Randomly selected 2.4.11
1/1	2.1.19	#56: Cannot write question based on computer read-out. Randomly selected 2.1.7
3	2.1.34	#66: Cannot write an operationally valid question. Randomly selected 2.1.3
3	2.2.40	#69: Cannot write an operationally valid question. Randomly selected 2.2.14
3	2.3.5	#70: K/A is oversampled. Randomly selected 2.4.49

3	2.1.27	#75: Cannot write an operationally valid question. Randomly selected 2.1.41
1/1	295016 AA2.05	#77: Cannot write an operationally valid question. Randomly selected 295031, EA2.04
1/1	EA2.02	#78: Cannot write SRO level question. Randomly selected EA2.01
1/2	295017	#83: Cannot write an operationally valid question. Randomly selected 500000
2/1	2.4.49	#88: Cannot write an SRO level question. Randomly selected 2.2.37
2/1	2.4.2	#90: Cannot write an operationally valid question. Randomly selected 2.2.42
2/2	290003 A2.04	#91: JPM already tests this subject (oversample). Randomly selected 290001 A2.02
2/2	2.4.46	#92: Cannot write an operationally valid question. Randomly selected 2.4.45
2/2	A2.02	#93: Operating scenario already tests this K/A (oversample). Randomly selected A2.03
3	2.3.15	#96: K/A is already selected for Tier 3 (oversample). Randomly selected 2.3.4
3	2.1.6	#100: K/A is tested in the operating test. Randomly selected 2.2.5.

Facility: Hope Creek Generating St	ation	Date of Examination: 8/2016		
Examination Level: RO 🛛 S	RO 🗌	Operating Test Number:1		
Administrative Topic (see Note)	Type Code*	Describe activity to be performed		
Conduct of Operations	N, R	ROA.1 Perform The Watchstanding Duties Of The Nuclear Control Room Operator		
Conduct of Operations	M, R	ROA.2 Complete A Sunday Shift Routine Log		
Equipment Control	N, R	ROA.3 Perform A Manual Tagout With SAP System Inoperable		
Radiation Control	D, P, S	ROA.4 Respond To An Abnormal Release Of Gaseous Radioactivity		
Emergency Plan				
NOTE: All items (five total) are required for SROs. RO applicants require only four items unless they are retaking only the administrative topics (which would require all five items).				
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or $Class(R)oom$ (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1 ; randomly selected)				

.

Facility: Hope Creek Generating Station Date of Examination:				
Examination Level: RO S	ro 🛛	Operating Test Number:1		
Administrative Topic (see Note)	Type Code*	Describe activity to be performed		
Conduct of Operations	D, R	SROA.1 Complete the Daily Surveillance Logs		
Conduct of Operations	N, R	SROA.2 Ensure The Operating Shift Is Adequately Manned		
Equipment Control	N, R	SROA.3 Perform WCD Review and Pre-Approval		
Radiation Control	D, P, S	SROA.4 Respond To An Abnormal Release Of Gaseous Radioactivity		
Emergency Plan	D, R	SROA.5 Utilize The ECG To Determine The Emergency Classification And/Or Reportability Of An Event And/Or Plant Condition		
NOTE: All items (five total) are required for SROs. RO applicants require only four items unless they are retaking only the administrative topics (which would require all five items).				
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (\leq 3 for ROs; \leq 4 for SROs & RO retakes) (N)ew or (M)odified from bank (\geq 1) (P)revious 2 exams (\leq 1; randomly selected)				

Control Room/In-Plant Systems Outline

Form ES-301-2

Facility: Hope Creek Generating Station Date of Examination: 8/2016						
Exam Level: RO SRO-I SRO-U COperating Test No: 1						
Control Room Systems:* 8 for RO; 7 for SRO-I; 2 or 3	for SRO-U					
System / JPM Title		Type Code*	Safety Function			
a. BF011 Exercise Control Rod, Respond To Uncou	a. BF011 Exercise Control Rod, Respond To Uncoupled Rod A, D, EN, S 1					
b. AE005 Place The First RFPT In Service		D, S	2			
c. AC008 Perform a Cooldown Using Bypass Valve	s	D, L, S	3			
d. BJ014 Place HPCI in Full Flow Test Operation		A, D, EN, L, S	4			
e. BC003 Respond To A Loss Of Shutdown Cooling	e. BC003 Respond To A Loss Of Shutdown Cooling D, L 5					
f. PB001 Shift A 4160 V 1E Bus To Alternate Feeder A, N, S 6						
g. GK002 Isolate the Control Room HVAC System A, EN, M, S 9						
h. EG002 Transfer TACS to the Standby SACS Loop D, S 8						
In-Plant Systems* (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)						
i. SB015 Transfer RPS Bus A/B Power From Alternate Source To RPS MG A, D Set						
j. BF002 Shift In-Service CRD Stabilizing Valves	j. BF002 Shift In-Service CRD Stabilizing Valves					
k. PN005 Respond To A Station Blackout		E, L, N	6			
 * All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all five SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room. 						
* Type Codes	* Type Codes Criteria for RO / SRO-I / SRO-U					
(A)Iternate path (C)ontrol room $4-6 / 4-6 / 2-3$ (D)irect from bank (E)mergency or abnormal in-plant (EN)gineered safety feature (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) $4-6 / 4-6 / 2-3$ (A)Iternate path $\leq 1 / \leq 1$ $\geq 1 / \geq 1$ $\leq 9 / \leq 8 / \leq 4$ $\geq 1 / \geq 1 / \geq 1$ $\geq 1 / \geq 1 / \geq 1$ (CN)gineered safety feature (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) $\geq 1 / \geq 1 / \geq 1$ $\leq 2 / \geq 2 / \geq 1$ 						

Control Room/In-Plant Systems Outline

Facility: Hope Creek Generating Station Date of Examination: 8/2016					
Exam Level: RO SRO-I SRO-U Operating Test No: 1					
Control Room Systems:* 8 for RO; 7 for SRO-I; 2 or 3 for	SRO-U	and and and and and			
System / JPM Title		Type Code*	Safety Function		
a. BF011 Exercise Control Rod, Respond To Uncoupled Rod A, D, EN, S 1					
b. AE005 Place The First RFPT In Service		D, S	2		
C. AC008 Perform a Cooldown Using Bypass Valves		D, L, S	3		
d. BJ014 Place HPCI in Full Flow Test Operation		A, D, EN, L, S	4		
e. BC003 Respond To A Loss Of Shutdown Cooling	e. BC003 Respond To A Loss Of Shutdown Cooling D, L 5				
f. PB001 Shift A 4160 V 1E Bus To Alternate Feeder	A, N, S	6			
g. GK002 Isolate the Control Room HVAC System	A, EN, M, S	9			
h.					
In-Plant Systems* (3 for RO); (3 for SRO-I); (3 or 2 for SF	RO-U)				
i. SB015 Transfer RPS Bus A/B Power From Alternate Source To RPS MG A, D Set			7		
j. BF002 Shift In-Service CRD Stabilizing Valves D, R			1		
k. PN005 Respond To A Station Blackout		E, L, N	6		
 * All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all five SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room. 					
* Type Codes	* Type Codes Criteria for RO / SRO-I / SRO-U				
(A)Iternate path $4-6 / 4-6 / 2-3$ (C)ontrol room $(D)irect from bank(D)irect from bank29 / \leq 8 / \leq 4(E)mergency or abnormal in-plant21 / \geq 1 / \geq 1(EN)gineered safety feature21 / \geq 1 / \geq 1(L)ow-Power / Shutdown21 / \geq 1 / \geq 1(N)ew or (M)odified from bank including 1(A)22 / \geq 2 / \geq 1(P)revious 2 exams\leq 3 / \leq 3 / \leq 2 (randomly selected)(R)CA21 / \geq 1 / \geq 1$					

Control Room/In-Plant Systems Outline

Form ES-301-2

Facility:	Hope Creek (Generating Statio	on		Date of Exam	ination: 8/	2016
Exam Level:	RO	SRO-I	SRO-U	\triangleleft	Operating Tes	st No:	1
Control Room	Systems:* 8 for	RO; 7 for SRO-	-l; 2 or 3 for	SRO-U			
	System / JPM Title Type Code* Safety Function						
a.							
b.							
с.							
d. BJ014 PI	ace HPCI in Ful	II Flow Test Ope	ration			A, D, EN, L, S	4
e. BC003 R	espond To A Lo	oss Of Shutdowr	n Cooling			D, L	5
f.							
g. GK002 Is	g. GK002 Isolate the Control Room HVAC System					A, EN, M, S	9
h.							
In-Plant Syste	ms* (3 for RO);	(3 for SRO-I); (3	3 or 2 for SR	:O-U)			
i.							
j. BF002 SI	j. BF002 Shift In-Service CRD Stabilizing Valves					D, R	1
k. PN005 R	espond To A St	ation Blackout				E, L, N	6
* All RC functio may o	 * All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all five SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room. 						
	* Туре С	odes			Criteria for RC) / SRO-I / SRO-I	J
 (A)Iternate path (C)ontrol room (D)irect from bank (E)mergency or abnormal in-plant (EN)gineered safety feature (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) (P)revious 2 exams (R)CA (S)imulator 				4-6 / 4-6 / <9 / <8 / >1 / >1 / >1 / >1 / >1 / >1 / >1 /	2-3 ≤ 4 ≥ 1 ≥ 1 (control room ≥ 1 ≤ 2 (randomly set ≥ 1	n system) elected)	

Appendix D

•--

Scenario Outline

Form ES-D-1

Facility:	Hope Creek Scer	ario No.: <u>1</u>	Op-Test No.: 2016 HC			
Examine	Examiners: Operators:					
Initial Co	onditions: <u>The scenari</u>	o begins with	the plant at approximately 88% power, with			
power a emerge	power ascension in progress following maintenance on RFPT AP101. Additionally, emergency diesel generator AG400 surveillance testing is in progress					
Turnova	r: The crow will remov		0 from service IAW Steps 5 / 13d -19 of HC OP-			
ST.KJ-0	001, and commence p	ower ascensi	on to 100% power in accordance with REMA.			
Event No.	Malf. No.	Event Type*	Event Description			
1	N/A	N (BOP)	Remove EDG from service (TS)			
2	N/A	R (ATC)	Raise reactor power using control rods			
3	CD033039	C (ALL)	Stuck control rod 30-39 (TS)			
4	PC07A TC16	C (ALL)	Seismic event (> OBE) causes EHC filter clogging			
5	AD02DO	C (ALL)	Safety Relief Valve F013D stuck open (TS)			
6	RP06	M (ALL)	1/2 core ATWS			
7	SL04A	C (ATC)	'A' SLC pump fails to auto start			
	SL01B		Trip of 'B' SLC pump			
8	CU11A	C (ATC)	RWCU fails to automatically isolate			
	CU11B					
9	TC07A	C (ALL)	Total loss of EHC, loss of bypass valves			
	ТС07В					
10	QQ20	C (BOP)	'A' RHR AP202 pump trip			
(N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor						

Appendix D

Scenario Outline

Form ES-D-1

•

Facility:	Facility: <u>Hope Creek</u> Scenario No.: <u>2</u> Op-Test No.: <u>2016 HC</u>						
Examine	Examiners: Operators:						
Initial Co	Initial Conditions: Reactor start up is in progress following a forced outage.						
Turnover: The crew will place remaining circulating water pumps in service IAW HC.OP- SO.DA-0001 step 5.1.28 through 5.2.10, and continue with reactor startup.							
Event No.	Malf. No.	Event Type*	Event Description				
1	N/A	N (BOP)	Place remaining circulating water pump in service				
2	N/A	R (ATC)	Raise power with control rods				
3	NM12D	I (ALL)	Recirculation system flow summer K607D failure (TS)				
4	ED09B1	C (ALL)	Loss of BD481 120 VAC 1E inverter (TS)				
5	MS04B	C (ALL)	Steam leak in steam tunnel – scram				
6	MS19B	C (ATC)	MSIV F022B isolation failure				
7	RH07D	M (ALL)	LOCA – RHR leak via pump suction, Loop D				
	HP08		HPCI failure				
8	PC06	C (ALL)	Suppression pool leak				
(N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor							

Appendix D

Scenario Outline

Form ES-D-1

Facility: <u>Hope Creek</u> Scenario No.: <u>3</u>			Op-Test No.: <u>2016 HC</u>	
Examiners:			Operators:	
Initial Co	onditions: 95% power			
Turnover: Ascend to 100% power IAW HC.OP-IO.ZZ-0006				
Event No.	Malf. No.	Event Type*	Event Description	
1	N/A	R (ATC)	Raise power with recirculation pumps	
2	N/A	C (ALL)	'A' Recirculation pump runaway (TS)	
3	IA03	C (BOP)	Primary containment instrument gas leak	
4	ED03	C (ALL)	Loss of 10A110 (TS)	
5	QQ02, QQ03, FW04B, FW04C	M (ALL)	Loss of feedwater with automatic scram failure on level 3	
6	HP03	C (BOP)	HPCI pump trip	
7	RR31A2	M (ALL)	LOCA	
8	R20E	I (BOP)	Failure of A channel of Core Spray and RHR logic	
(N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor				

. . ---