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JUN 30 2017

Docket Nos.: 52-025
52-026

ND-17-1168
10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3 and Unit 4
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load
Item 2.5.01.02a [Index Number 506]

Ladies and Gentlemen:

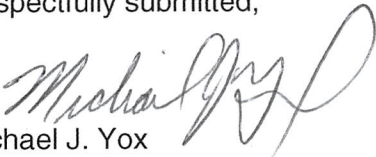
Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of June 21, 2017, Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4 Uncompleted Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Item 2.5.01.02a [Index Number 506] has not been completed greater than 225-days prior to initial fuel load. The Enclosure describes the plan for completing ITAAC 2.5.01.02a [Index Number 506]. Southern Nuclear Operating Company will, at a later date, provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI-08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(3) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact David Woods at 706-848-6903.

Respectfully submitted,


Michael J. Yox
Regulatory Affairs Director Vogtle 3 & 4

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Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4
Completion Plan for Uncompleted ITAAC 2.5.01.02a [Index Number 506]

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File AR.01.02.06

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**Southern Nuclear Operating Company
ND-17-1168
Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 3 and Unit 4
Completion Plan for Uncompleted ITAAC 2.5.01.02a [Index Number 506]**

ITAAC Statement

Design Commitment

2.a) The DAS provides an automatic reactor trip on low wide-range steam generator water level, or on low pressurizer water level, or on high hot leg temperature, separate from the PMS.

Inspections/Tests/Analyses

Electrical power to the PMS equipment will be disconnected and an operational test of the as-built DAS will be performed using real or simulated test signals.

Acceptance Criteria

The generator field control relays (contained in the control cabinets for the rod drive motor-generator sets) open after the test signal reaches the specified limit.

ITAAC Completion Description

Testing is performed in accordance with the Unit 3 and Unit 4 preoperational test procedures SV3-DAS-T1P-501 and SV4-DAS-T1P-501 (References 1 and 2, respectively) to verify that the Diverse Actuation System (DAS) provides an automatic reactor trip on low wide-range steam generator water level, or on low pressurizer water level, or on high hot leg temperature, separate from the Protection and Safety Monitoring System (PMS).

Electrical power to the PMS equipment is removed and an operational test of the as-built DAS is performed using simulated test signals. Initially, the generator field control relays for the control rod motor-generator (MG) sets are verified to be closed through local verification. The DAS instruments for wide range steam generator level, wide range pressurizer level, and hot leg temperature have simulated signals generated by use of test equipment. When the required setpoints and logic for instrumentation actuation have been met, the generator field control relays of the control rod motor-generator sets are locally verified to trip open. This testing verifies the combinations of involved instrumentation, upon exceeding the actuation setpoint, will generate a reactor trip. The DAS processor cabinet 1 is utilized to confirm reactor trip status.

The reports documenting the Unit 3 and Unit 4 preoperational test results, SV3-DAS-T2R-501 and SV4-DAS-T2R-501 (References 3 and 4, respectively), confirm the generator field control relays (contained in the control cabinets for the rod drive motor-generator sets) open after the test signal reaches the specified limit.

References 1, 2, 3, and 4 are available for NRC inspection as part of the ITAAC 2.5.01.02a Completion Package (Reference 5).

List of ITAAC Findings

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

References (available for NRC inspection)

1. SV3-DAS-T1P-501, "Diverse Actuation System Preoperational Test Procedure"
2. SV4-DAS-T1P-501, "Diverse Actuation System Preoperational Test Procedure"
3. SV3-DAS-T2R-501, "Diverse Actuation System Preoperational Test Results Report"
4. SV4-DAS-T2R-501, "Diverse Actuation System Preoperational Test Results Report"
5. ITAAC 2.5.01.02a Completion Package
6. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"