Commonwealth Edison pany Dresden Generating Station 6500 North Dresden Road Morris, II, 60450 Tel 815-942-2920

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July 5, 1995

TPJLTR 95-0075

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Licensee Event Report 95-018, Docket 50-237 is being submitted as required by Technical Specification 6.6 and 10CFR50.73(a)(2)(i) and 10CFR50.73(a)(2)(ii).

Sincerely

Thomas P./Joyce Site Vice President

TPJ/:pt

Enclosure

cc: J. Martin, Regional Administrator, Region III NRC Resident Inspector's Office File/NRC File/Numerical

TPJ95\0075.95

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NRC FOR (5-92)	IRC FORM 366 U.S. NUCLEAR REGULAT((5-92)					REGULATO	RY COM	ISSION	APPROVED BY ONB NO. 3150-0104 EXPIRES 5/31/95							
	LICENSEE EVENT REPORT (LER)								ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.							
FACILIT	FACILITY NAME (1) Dresden Nuclear Power Station, Unit 2						DOCKET NUMBER (2) 05000237			PAGE (3) 1 OF 3						
TITLE (4) Type	Ban	d C I	Leak	age Limit	Exce	ede	d Due	to E	хсевв	ive Le	akage Past	: HPCI	Che	ck V	alve
EVEN	T DATE	(5)			ER NUMBER (6)		REPO	RT DATE	(7)	T	OTHER FACIL	ITIES IN	OL VE	(8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVIS	ION ER	MONTH	DAY	YEAR	FACILIT	CILITY NAME		DOCKET NUMBER		
06	10	95	95		018	00)	07	10	95	FACILIT	ACILITY NAME		DOCKET NUMBER		
OPERA	OPERATING N THIS REPORT IS SUBMITTED PURSUANT TO THE REQU				REQUIR	EMENTS	OF 10 CF	R§: (Check o	one or mo	re) (11)					
MODE (9) 20.2201(b)			L					50.73(8)(2)(11)			5./10	D)				
POM	POWER		20	20.2203(a)(1)			20.2203(a)(3)(11)				50.73(a)(2)(1	v)	/3./1(c)			
LEVEL	(10)			.2203	(a)(2)(i)			20.2203	(a)(4)			50.73(a)(2)(v	()	ں	THER	
			20	.2203	(a)(2)(ii)			50.36(c)(1)			50.73(a)(2)(v	/ii)	(Spe	City	1n halou
			20).2203	(a)(2)(iii)			50.36(c)(2)			50.73(a)(2)(v	/iii)(A)	and	in Te	xt,
			20).2203	(a)(2)(iv)		X	50.73(a)(2)(i)		50.73(a)(2)(v	/iii)(B)	NRC Form 366A)		
			20	.2203	5(a)(2)(v)		X	50.73(a)(2)(i	i)		50.73(a)(2)()	0			
						LICENS	EE (CONTACT	FOR THI	S LER	(12)					
NAME M. McGivern, Local Leak Rate Test Coordinator Ext. 2526 (815) 942-2920					Area (O	;ode)										
			0	MPLE	TE ONE LINE FO	OR EACH	CO)	PONENT I	FAILURE	DESCRI	BED IN 1	HIS REPORT (1	3)			
CAUSE	SYSTE	EM C	OMPONE	т	MANUFACTURER	REPOR TO N	TABL	LE S	C	AUSE	SYSTEM	COMPONENT	MANUFAC	TURE	RE	PORTABLE O NPRDS
х	BJ		ISV		C283	Ye	98									
	SUPPLEMENTAL REPORT EXPECTED (14)							E	PECTED	MONTH		DAY	YEAR			
X YES (1f	X YES (If yes, complete EXPECTED SUBMISSION DATE).						60		D/	SUBMISSION DATE (15) 11 1		13	95			

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

At approximately 0830, on June 10, 1995, with Unit 2 shutdown for Refuel Outage D2R14, the performance of Dresden Technical Surveillance (DTS) 1600-01, Local Leak Rate Testing Of Primary Containment Isolation Valves, identified the High Pressure Coolant Injection (HPCI) System Turbine Exhaust to Suppression Pool Check Valve 2-2301-45 to be leaking more than the test equipment could measure. When the valve's leakage was added to the existing maximum pathway leakage rate, the maximum pathway leakage rate limit for Type B and C primary containment leakage, 488.452 standard cubic feet per hour (scfh) (0.6L), as listed in Technical Specification 3.7.A.2.b.(2)(a) was exceeded. The safety significance of the leakage past the 2-2301-45 was considered to be minimal since the additional leakage out of containment, on a minimum pathway basis, was 0 scfh from the inboard isolation Stop Check Valve 2-2301-74 and would not cause the maximum off-site dose rates established in 10 CFR 100 to be exceeded. The check valve will be removed, inspected, replaced and Local Leak Rate Tested prior to unit startup. A supplement will be submitted to report the reason for this valve and any other valve failures during D2R14 and the corrective actions taken.

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•	NRC FORM 366A (5-92)	U.S. NUCLEAR RE	GULATORY COMMISSION		APPROVED BY O EXPIRE	NB NO. 315 S 5/31/95	0-0104
	-	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	'R)	ESTIMATED BURDEN PER RESPONSE TO C THIS INFORMATION COLLECTION REQUEST: FORWARD COMMENTS REGARDING BURDEN E THE INFORMATION AND RECORDS MANAGEM (MNBB 7714), U.S. NUCLEAR REGULATORY WASHINGTON, DC 20555-0001, AND TO TH REDUCTION PROJECT (3150-0104), MANAGEMENT AND BUDGET, WASHINGTON, DC			
		FACILITY NAME (1)	DOCKET MUMBER (2)		LER NUMBER (6)		PAGE (3)
	Duesden Nu	clear Power Station, Unit 2	05000237	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 07 2
	DIERGEN NU			95	018	00	2 UF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT IDENTIFICATION:

Type B and C Leakage Limit Exceeded Due to Excessive Leakage Past HPCI Check Valve

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 2	2		Event Date:	06/10/95	Event	Time:	0830	hrs
Reactor	Mode:	N	Mode Name:	Refuel	Power	Level:	0%	
Reactor	Coolant	: Svstem	Pressure: 0	psig				

B. DESCRIPTION OF EVENT:

At approximately 0830, on June 10, 1995, with Unit 2 shutdown for Refuel Outage D2R14, the performance of Dresden Technical Surveillance (DTS) 1600-01, Local Leak Rate Testing Of Primary Containment Isolation Valves, identified the HPCI Turbine Exhaust to Suppression Pool Check Valve 2-2301-45 to be leaking more than the test equipment could measure. When the valve's leakage was added to the existing maximum pathway leakage rate, the maximum pathway leakage rate limit for Type B and C primary containment leakage, 488.452 scfh (0.6L), as listed in Technical Specification 3.7.A.2.b.(2)(a) was exceeded.

The Unit Supervisor was notified of the event and a Performance Improvement Form (PIF) was written to report a condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT:

This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(i) which requires the reporting of any operation or condition prohibited by the plant's Technical Specifications.

This LER is also submitted pursuant to 10 CFR 50.73(a)(2)(ii) which requires reporting any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded.

The dual-disk HPCI Turbine Exhaust Check Valve 2-2301-45 will be removed, inspected, replaced, and Local Leak Rate Tested prior to unit start up.

A supplement to this LER will be submitted to document the cause of the check valve's LLRT failure.

D. SAFETY ANALYSIS:

The safety significance of the leakage past the 2-2301-45 was considered to be minimal since the additional leakage out of containment, on a minimum pathway basis, was 0 scfh from the inboard isolation Stop Check Valve 2-2301-74 and would not cause the maximum off-site dose rates established in 10 CFR 100 to be exceeded.

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NRC FORM 366A (5-92)	FORN 366A U.S. NUCLEAR REGULATORY COMMISSION 92)			APPROVED BY ONB NO. 3150-0104 EXPIRES 5/31/95				
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	FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)		PAGE (3)	
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Dresden Nuc	tear Power Station, (05000237	95	018	00	5 OF 3	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

E. CORRECTIVE ACTIONS:

Nuclear Tracking System (NTS) tracking code numbers are identified in the text as (XXX-XXX-XX-XXXXX).

The HPCI Turbine Exhaust Check Valve 2-2301-45 will be removed, inspected, replaced, and Local Leak Rate Tested prior to unit start up. (NTS #237-180-95-01801)

An LER supplement will be submitted which contains the cause of and the repairs performed for all D2R14 valve LLRT failures as well as the results of the asleft LLRTs. (NTS #237-180-95-01802)

- F. PREVIOUS OCCURRENCES:
 - LER/Docket Numbers Title

95-011/0500249 Type B and C Leakage Limit Exceeded Due to Excessive Leakage Past HPCI Check Valve

- 94-022/0500237 Type B and C Leakage Limit Exceeded Due to Worn Seating Surface of HPCI Check Valve
- 91-007/0500249 Type B and C Containment Local Leak Rate Testing Limit Exceeded Due to HPCI Turbine Exhaust Check Valve Leakage
- 89-009/0500249 Local Leak Rate Testing "As Found" limit Exceeded Due to leakage From Primary Containment Valves

G. COMPONENT FAILURE DATA:

<u>Manufacturer</u>	Nomenclature	<u>Model Number</u>	<u>Mfg. Part Number</u>		
C & S Valve Co.	HPCI Turbine Exhaust to Suppression Pool Check Valve 2-2301-45	N/A	N/A		

An LER supplement will be submitted with the results of an industry wide Nuclear Plant Reliability Data System (NPRDS) data base search of similar valve failures.