

Commonwealth Edison Company
Dresden Generating Station
6500 North Dresden Road
Morris, IL 60450
Tel 815-942-2920

ComEd

July 5, 1995

TPJLTR 95-0075

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Licensee Event Report 95-018, Docket 50-237 is being submitted as required by Technical Specification 6.6 and 10CFR50.73(a)(2)(i) and 10CFR50.73(a)(2)(ii).

Sincerely,



Thomas P. Joyce
Site Vice President

TPJ/:pt

Enclosure

cc: J. Martin, Regional Administrator, Region III
NRC Resident Inspector's Office
File/NRC
File/Numerical

TPJ95\0075.95

9507130009 950710
PDR ADOCK 05000237
S PDR

100004

TEJ221

NRC FORM 366 (5-92)		U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95							
LICENSEE EVENT REPORT (LER)								ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.				
FACILITY NAME (1) Dresden Nuclear Power Station, Unit 2					DOCKET NUMBER (2) 05000237			PAGE (3) 1 OF 3				
TITLE (4) Type B and C Leakage Limit Exceeded Due to Excessive Leakage Past HPCI Check Valve												
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER		
06	10	95	95	-- 018 --	00	07	10	95	None			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)										
N		20.2201(b)		20.2203(a)(3)(i)		50.73(a)(2)(iii)		73.71(b)				
POWER LEVEL (10)		20.2203(a)(1)		20.2203(a)(3)(ii)		50.73(a)(2)(iv)		73.71(c)				
000		20.2203(a)(2)(i)		20.2203(a)(4)		50.73(a)(2)(v)		OTHER				
		20.2203(a)(2)(ii)		50.36(c)(1)		50.73(a)(2)(vii)		(Specify in Abstract below and in Text, NRC Form 366A)				
		20.2203(a)(2)(iii)		50.36(c)(2)		50.73(a)(2)(viii)(A)						
		20.2203(a)(2)(iv)		X 50.73(a)(2)(i)		50.73(a)(2)(viii)(B)						
		20.2203(a)(2)(v)		X 50.73(a)(2)(ii)		50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)												
NAME M. McGivern, Local Leak Rate Test Coordinator Ext. 2526						TELEPHONE NUMBER (Include Area Code) (815) 942-2920						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)												
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		
X	BJ	ISV	C283	Yes								
SUPPLEMENTAL REPORT EXPECTED (14)												
X	YES (If yes, complete EXPECTED SUBMISSION DATE).			NO	EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
							11	13	95			

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

At approximately 0830, on June 10, 1995, with Unit 2 shutdown for Refuel Outage D2R14, the performance of Dresden Technical Surveillance (DTS) 1600-01, Local Leak Rate Testing Of Primary Containment Isolation Valves, identified the High Pressure Coolant Injection (HPCI) System Turbine Exhaust to Suppression Pool Check Valve 2-2301-45 to be leaking more than the test equipment could measure. When the valve's leakage was added to the existing maximum pathway leakage rate, the maximum pathway leakage rate limit for Type B and C primary containment leakage, 488.452 standard cubic feet per hour (scfh) (0.6L₁), as listed in Technical Specification 3.7.A.2.b.(2)(a) was exceeded. The safety significance of the leakage past the 2-2301-45 was considered to be minimal since the additional leakage out of containment, on a minimum pathway basis, was 0 scfh from the inboard isolation Stop Check Valve 2-2301-74 and would not cause the maximum off-site dose rates established in 10 CFR 100 to be exceeded. The check valve will be removed, inspected, replaced and Local Leak Rate Tested prior to unit startup. A supplement will be submitted to report the reason for this valve and any other valve failures during D2R14 and the corrective actions taken.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Dresden Nuclear Power Station, Unit 2	05000237	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		95	-- 018 --	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT IDENTIFICATION:

Type B and C Leakage Limit Exceeded Due to Excessive Leakage Past HPCI Check Valve

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 2 Event Date: 06/10/95 Event Time: 0830 hrs.
Reactor Mode: N Mode Name: Refuel Power Level: 0%
Reactor Coolant System Pressure: 0 psig

B. DESCRIPTION OF EVENT:

At approximately 0830, on June 10, 1995, with Unit 2 shutdown for Refuel Outage D2R14, the performance of Dresden Technical Surveillance (DTS) 1600-01, Local Leak Rate Testing Of Primary Containment Isolation Valves, identified the HPCI Turbine Exhaust to Suppression Pool Check Valve 2-2301-45 to be leaking more than the test equipment could measure. When the valve's leakage was added to the existing maximum pathway leakage rate, the maximum pathway leakage rate limit for Type B and C primary containment leakage, 488.452 scfh (0.6L), as listed in Technical Specification 3.7.A.2.b.(2)(a) was exceeded.

The Unit Supervisor was notified of the event and a Performance Improvement Form (PIF) was written to report a condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT:

This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(i) which requires the reporting of any operation or condition prohibited by the plant's Technical Specifications.

This LER is also submitted pursuant to 10 CFR 50.73(a)(2)(ii) which requires reporting any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded.

The dual-disk HPCI Turbine Exhaust Check Valve 2-2301-45 will be removed, inspected, replaced, and Local Leak Rate Tested prior to unit start up.

A supplement to this LER will be submitted to document the cause of the check valve's LLRT failure.

D. SAFETY ANALYSIS:

The safety significance of the leakage past the 2-2301-45 was considered to be minimal since the additional leakage out of containment, on a minimum pathway basis, was 0 scfh from the inboard isolation Stop Check Valve 2-2301-74 and would not cause the maximum off-site dose rates established in 10 CFR 100 to be exceeded.

NRC FORM 366A (5-92)		U.S. NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95			
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION				ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.			
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)		PAGE (3)	
Dresden Nuclear Power Station, Unit 2		05000237		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 3
				95	-- 018 --	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

E. CORRECTIVE ACTIONS:

Nuclear Tracking System (NTS) tracking code numbers are identified in the text as (XXX-XXX-XX-XXXXX).

The HPCI Turbine Exhaust Check Valve 2-2301-45 will be removed, inspected, replaced, and Local Leak Rate Tested prior to unit start up. (NTS #237-180-95-01801)

An LER supplement will be submitted which contains the cause of and the repairs performed for all D2R14 valve LLRT failures as well as the results of the as-left LLRTs. (NTS #237-180-95-01802)

F. PREVIOUS OCCURRENCES:

<u>LER/Docket Numbers</u>	<u>Title</u>
95-011/0500249	Type B and C Leakage Limit Exceeded Due to Excessive Leakage Past HPCI Check Valve
94-022/0500237	Type B and C Leakage Limit Exceeded Due to Worn Seating Surface of HPCI Check Valve
91-007/0500249	Type B and C Containment Local Leak Rate Testing Limit Exceeded Due to HPCI Turbine Exhaust Check Valve Leakage
89-009/0500249	Local Leak Rate Testing "As Found" limit Exceeded Due to leakage From Primary Containment Valves

G. COMPONENT FAILURE DATA:

<u>Manufacturer</u>	<u>Nomenclature</u>	<u>Model Number</u>	<u>Mfg. Part Number</u>
C & S Valve Co.	HPCI Turbine Exhaust to Suppression Pool Check Valve 2-2301-45	N/A	N/A

An LER supplement will be submitted with the results of an industry wide Nuclear Plant Reliability Data System (NPRDS) data base search of similar valve failures.