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Dresden Generating Station
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ComEd

June 26, 1995

TPJLTR 95-0068

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Licensee Event Report 95-011, Docket 50-249 is being submitted as required by Technical Specification 6.6, 10CFR50.73(a)(2)(i) and 10CFR50.73(a)(2)(ii).

Sincerely,

Thomas P. Joyce
Site Vice President

TPJ/MM:pt

Enclosure

cc: J. Martin, Regional Administrator, Region III
NRC Resident Inspector's Office
File/NRC
File/Numerical

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NRC FORM 366 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Dresden Nuclear Power Station, Unit 3	05000249	1 OF 3

TITLE (4)
Type B and C Leakage Limit Exceeded Due to Excessive Leakage Past HPCI Check Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	29	95	95	-- 011 --	00	06	28	95	None	
									FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	N
POWER LEVEL (10)	000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)

	20.2201(b)		20.2203(a)(3)(i)		50.73(a)(2)(iii)		73.71(b)
	20.2203(a)(1)		20.2203(a)(3)(ii)		50.73(a)(2)(iv)		73.71(c)
	20.2203(a)(2)(i)		20.2203(a)(4)		50.73(a)(2)(v)		OTHER
	20.2203(a)(2)(ii)		50.36(c)(1)		50.73(a)(2)(vii)		(Specify in Abstract below and in Text, NRC Form 366A)
	20.2203(a)(2)(iii)		50.36(c)(2)		50.73(a)(2)(viii)(A)		
	20.2203(a)(2)(iv)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(B)		
	20.2203(a)(2)(v)	X	50.73(a)(2)(ii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
M. McGivern, Local Leak Rate Test Coordinator Ext. 2526	(815) 942-2920

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	BJ	ISV	C283	Yes						

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE).	NO		11	13	95

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

At approximately 1600, on May 29, 1995 with Unit 3 shutdown for maintenance, the performance of Dresden Technical Surveillance (DTS) 1600-01, Local Leak Rate Testing Of Primary Containment Isolation Valves, identified the High Pressure Coolant Injection (HPCI) System [BJ] Turbine Exhaust to Suppression Pool Check Valve 3-2301-45 to be leaking more than the test equipment could measure. When the valve's leakage was added to the existing maximum pathway leakage rate, the maximum pathway leakage rate limit for Type B and C primary containment leakage, 488.452 standard cubic feet per hour (scfh) ($0.6L_a$), as listed in Technical Specification 3.7.A.2.b.(2)(a) was exceeded. The safety significance of the leakage past the 3-2301-45 was considered to be minimal since the additional leakage out of containment, on a minimum pathway basis, was 14.39 scfh from the inboard isolation Stop Check Valve 3-2301-74 (LLRT performed on 9/2/94) and would not cause the maximum off-site dose rates established in 10 CFR 100 to be exceeded. The check valve will be removed, inspected, replaced and Local Leak Rate Tested prior to unit startup. A supplement will be submitted to report the reason for valve failure and the corrective actions taken.

EVENT IDENTIFICATION:

Type B and C Leakage Limit Exceeded Due to Excessive Leakage Past HPCI Check Valve

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 3 Event Date: 05/29/95 Event Time: 1600 hrs.

Reactor Mode: N Mode Name: Shutdown Power Level: 0%

Reactor Coolant System Pressure: 0 psig

B. DESCRIPTION OF EVENT:

At approximately 1600, on May 29, 1995 with Unit 3 shutdown for maintenance, the performance of Dresden Technical Surveillance (DTS) 1600-01, Local Leak Rate Testing Of Primary Containment Isolation Valves, identified the HPCI Turbine Exhaust to Suppression Pool Check Valve 3-2301-45 to be leaking more than the test equipment could measure. When the valve's leakage was added to the existing maximum pathway leakage rate, the maximum pathway leakage rate limit for Type B and C primary containment leakage, 488.452 scfh ($0.6L_a$), as listed in Technical Specification 3.7.A.2.b.(2)(a) was exceeded.

To verify that corrective actions previously undertaken to prevent LLRT failures were sufficient, leak rate tests were to be performed on an accelerated schedule of every six months. This first six month leak rate test resulted in an LLRT failure.

The Unit Supervisor was notified of the event and a Performance Improvement Form (PIF) was written to report a condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT:

This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(i) which requires the reporting of any operation or condition prohibited by the plant's Technical Specifications.

This LER is also submitted pursuant to 10 CFR 50.73(a)(2)(ii) which requires reporting any event or condition that resulted in the condition of the nuclear power plant, including its principal safety barriers, being seriously degraded.

The dual-disk HPCI Turbine Exhaust Check Valve 3-2301-45 will be removed, inspected, replaced and Local Leak Rate Tested prior to unit startup.

The root cause of the valve failure, and corrective actions to prevent future recurrence of failures, will be thoroughly reviewed prior to startup.

A supplement to this LER will be submitted to document the cause of the check valve's LLRT failure.

D. SAFETY ANALYSIS:

The safety significance of the leakage past the 3-2301-45 was considered to be minimal since the additional leakage out of containment, on a minimum pathway basis, was 14.39 scfh from the inboard isolation Stop Check Valve 3-2301-74 (LLRT performed on 9/2/94) and would not cause the maximum off-site dose rates established in 10 CFR 100 to be exceeded.

E. CORRECTIVE ACTIONS:

Nuclear Tracking System (NTS) tracking code numbers are identified in the text as (XXX-XXX-XX-XXXXX).

The HPCI Turbine Exhaust Check Valve 3-2301-45 will be replaced and Local Leak Rate Tested prior to startup. (NTS #249-180-95-01101)

An LER supplement will be submitted which contains the repairs performed and the results of the as-left LLRT. (NTS #249-180-95-01102)

The root cause of the valve failure, and corrective actions to prevent future recurrence of failures, will be thoroughly reviewed prior to startup. (NTS #249-180-95-01103)

F. PREVIOUS OCCURRENCES:

LER/Docket Numbers	Title
94-022/0500237	Type B and C Leakage Limit Exceeded Due to Worn Seating Surface of HPCI Check Valve
91-007/0500249	Type B and C Containment Local Leak Rate Testing Limit Exceeded Due to HPCI Turbine Exhaust Check Valve Leakage
89-009/0500249	Local Leak Rate Testing "As Found" limit Exceeded Due to leakage From Primary Containment Valves

G. COMPONENT FAILURE DATA:

An LER supplement will be submitted with the results of an industry wide Nuclear Plant Reliability Data System (NPRDS) data base search of similar valve failures along with specific failure information for the check valve obtained from valve disassembly.