



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

August 11, 1994

Mr. William T. Russell, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Dresden Nuclear Power Station Units 2 and 3
LaSalle Nuclear Power Station Units 1 and 2
Quad Cities Nuclear Power Station Units 1 and 2
Generic Letter 92-01, Supplement 1, "Reactor Vessel Structural Integrity"
NRC Docket Nos. 50-237/249, 50-254/265 and 50-373/374

- References:
- (a) J. Stang letter to D. Farrar, dated April 14, 1994.
 - (b) D.J. Chrzanowski to NRC, "Additional Information Regarding Generic Letter 92-01 revision 1, "Reactor Vessel Structural Integrity,"" September 24, 1993.
 - (c) GE Report MDE-89-0786, DRF A00-02764, "Flux Wire Dosimeter Evaluation for LaSalle Nuclear Power Station, Unit 1," July 1986.
 - (d) GE Report SASR 87-59, DRF A00-02764, "Flux Wire Dosimeter Evaluation for LaSalle Nuclear Power Station, Unit 2," October 1987.

Mr. Russell:

The Reference (a) letter required Commonwealth Edison (ComEd) to confirm commitment and applicability of the BWR Owners Group efforts to resolve reactor vessel structural issues, and to verify the information contained in an NRC database. The following is ComEd's response to the Reference (a) request.

Initial RT_{NDT} Values

ComEd is committed to the BWROG effort for establishment of initial RT_{NDT} values for the non-Linde 80 SAW material of Quad Cities Unit 2 and LaSalle Unit 1 and 2, and the plate material of Dresden Unit 2 and 3, Quad Cities Unit 1 and 2, and LaSalle Unit 1 and 2.

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A description of the data used to establish an initial RT_{NDT} value of $-5^{\circ}F$ for the Linde 80 SAW material of Quad Cities Unit 1 is provided in Reference (b).

Low Upper Shelf Energy (USE)

The applicability of topical report NEDO-32205-A, Revision 1 has been confirmed for Dresden Units 2 and 3, Quad Cities Units 1 and 2, and LaSalle Units 1 and 2. Material surveillance data for Dresden Units 2 and 3 and Quad Cities Units 1 and 2 were evaluated as specified in Appendix B of the NEDO report. No material surveillance data has yet been obtained for LaSalle Units 1 and 2.

ComEd requests NRC approval of topical report NEDO-32205-A, Revision 1 for demonstrating compliance with 10CFR50, Appendix G, Paragraph IV.A.1.

Verification of NRC Database

Summary File for Pressure-Temperature Limits

The fifth column on every page, " IRT_{NDT} ," contains a footnote 1, "Additional information required to confirm value." It is not clear to ComEd what the basis was for applying the footnote to individual entries. In general it appears that the footnote applies when the IRT_{NDT} value is from GE methodology ($CVN_{T50} - 60^{\circ}F$) instead of from drop weight testing, but this is not always the case. We request clarification on the basis for the footnote and on what information is required to confirm the value.

In the fifth column, " IRT_{NDT} ," for Dresden Unit 2 and 3 and Quad Cities Unit 1, for the SAW material the IRT_{NDT} is $-5^{\circ}F$ based on Reference (b).

In the fourth column of each page, "ID Neut. Fluence at EOL/EFPY," for LaSalle Units 1 and 2, the ID neutron fluence reported for end-of-life is actually the 1/4T neutron fluence. The upper bound ID neutron fluence values are $6.1E17$ and $6.4E17$ for LaSalle Units 1 and 2, based on References (c) and (d), respectively. It should be noted that the "Reference" at the end of the tables is incorrect in that fluence data for LaSalle Units 1 and 2 were not provided in the July 1, 1992 letter from M.A. Jackson (CECo) to T.E. Murley (USNRC).

Summary File for Upper Shelf Energy

The fourth column, "Material Type," requires the following corrections:

Dresden 2: Change "A302B" to A302B Mod.
 Change "Flux unknown" for SAW to Linde 80.

Dresden 3 and Quad Cities Unit 1 and 2: Change "Flux unknown" for SAW to Linde 80.

Quad Cities 1: Change "Flux unknown" for SAW to Linde 80.


Quad Cities 2: Change "Flux unknown" for SAW to Linde 124.

The sixth column, "1/4T Neutron Fluence at EOL" for LaSalle Units 1 and 2 should be changed to 3.9E17 and 4.2E17, based on references (c) and (d), respectively.

To the best of my knowledge and belief, the information contained herein is true and correct. In some respect, this information is not based on my personal knowledge, but upon information furnished by other Commonwealth Edison and contractor employees. Such information has been reviewed in accordance with company practice, and I believe it to be reliable.

If there are any questions concerning this matter, please contact this office.

Respectfully,


Peter L. Piet
Nuclear Licensing Administrator

cc: J.B. Martin, Regional Administrator - RIII
 A. Gody, Jr. - NRR
 J. Stang, Project Manager - NRR
 C. Patel, Project Manager - NRR
 M. Leach, Senior Resident Inspector - Dresden
 C. Miller, Senior Resident Inspector - Quad Cities
 B. Brochman, Senior Resident Inspector - LaSalle
 Office of Nuclear Facility Safety - IDNS