

**Commonwealth Edison** Dresden Nuclear Power Station 6500 North Dresden Road Morris, Illinois 60450 Telephone 815/942-2920

August 11, 1994

GFSLTR 94-0266

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Licensee Event Report 94-014, Docket 50-249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10CFR50.73(a)(2)(iv).

Sincerely

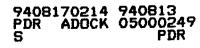
Gary F() Spedl Station Manager Dresden Station

GFS/JW:cfq

Enclosure

cc: J. Martin, Regional Administrator, Region III
NRC Resident Inspector's Office
File/NRC
File/Numerical

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EXPIRES 5/31/95         EXPIRES 5/31/95 <th>-</th> <th></th>	-																
LICENSEE EVENT REPORT (LER)           ESTIMATED BURGEN PER RESPONSE TO COMPLY WI THIS INFORMATION CALLECTION REQUESTS 50.0 mm REDUCTION AND RECORD STATUS           FACILITY NAME (1) Dresden Nuclear Power Station, Unit 3           DOCKT NUMBER (2) DOCKT NUMBER (2) D		NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION (5-92)															
Dresden Nuclear Power Station, Unit 3         05000249         1 OF 3           TITLE (4) Unit 3 Torus/Reactor Building Relief Valves Cycled due to Personnel Error         TOP 3           EVENT DATE (5)         LER NUMBER (6)         REPORT DATE (7)         OTHER FACILITIES INVOLVED (8)           MONTH         DAY         YEAR         YEAR         SEQUENTIAL NUMBER         REVISION NUMBER         MONTH         DAY         YEAR         DOCKET NUMBER         DOCKET NUMBER           07         14         94         94          014          00         08         13         94         FACILITY NAME         DOCKET NUMBER           0PREATING MODE (9)         N         THIS REPORT IS SUBNITED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)         TOCKET NUMBER         DOCKET NUMBER           POWER LEVEL (10)         000         20.2203(a)(2)(i)         20.2203(a)(3)(i)         X 50.73(a)(2)(vi)         73.71(c)           Q20.2203(a)(2)(i)         20.2203(a)(2)(i)         20.2203(a)(2)(i)         X 50.73(a)(2)(vii)         Mattract below and in Text, 20.2203(a)(2)(vi)         S0.36(c)(2)         S0.73(a)(2)(vii)(h)         Mattract below and in Text, 20.2203(a)(2)(v)         S0.73(a)(2)(vii)(h)         Mattract below and in Text, 20.2203(a)(2)(v)         S0.73(a)(2)(vii)(h)         Mattract below and in Text, 20.2203(a)(2)(v)         S0.73(a)(2)(v										ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104). OFFICE OF							
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On July 14, 1994, with Unit 3 in a refueling outage, the Torus Vacuum Breakers, 1601-20A and 1601-20B, cycled due to negative pressure in the suppression chamber. The cause of the event was due to closing the torus hatches with the Drywell and Torus Purge fan in operation while aligned to the torus only. The vacuum breakers began to cycle when the torus reached approximately -0.46 pounds per square inch differential (psid) between the reactor building and the torus. The Technical Specification requires the vacuum breakers operate at -0.5 psid between the reactor building and torus. The Unit 3 Nuclear Station Operator (NSO) was alerted by actuation of the "TORUS VACUUM RELIEF VALVE NOT CLOSED" annunciator alarm, and he observed the vacuum breakers cycling. Also, the Mechanical Maintenance Department personnel, responsible for closing the last torus hatch, observed the vacuum breakers cycling and called the control room to inform the operator. The Unit 3 NSO closed the 18 inch torus purge suction valve, 1601-60, and the torus vacuum breakers stopped cycling within approximately 20 seconds.

NRC FORM 366A U.S. NUCLEA (5-92)	U.S. NUCLEAR REGULATORY COMMISSION						
LICENSEE EVENT REPORT TEXT CONTINUATION							
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)	)	PAGE (3)		
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Dresden Nuclear Power Station, Unit 3	05000249	94	014	00	2 OF 3		

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT IDENTIFICATION:

Unit 3 Torus/Reactor Building Relief Valves Cycled due to Personnel Error

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: 3	Event Date: 07/14/94	Event Time: 0346
Reactor Mode: Shutde	own Mode Name: Refuel	Power Level: 0%
Reactor Coolant Syste	em Pressure: 0 psig	

## B. DESCRIPTION OF EVENT:

On July 13 1994, with Unit 3 in a refueling outage, the torus vacuum breakers, 1601-20A and 1601-20B, cycled due to negative pressure in the torus. At the time of this event, the torus and Drywell Purge Fans were aligned to the torus only to ensure adequate ventilation for personnel which had been working in the torus.

On July 13, 1994, at 1800 hours, the Shift Outage Manager (SOM) directed the Mechanical Maintenance Department (MMD) supervisor to close the Unit 3 torus hatches because all torus work was completed. At approximately 2200 hours, the MMD supervisor dispatched a crew to bolt the torus hatches. He notified the SOM that the hatches would be secured. The SOM did not inform the NSO or the Shift Control Room Engineer (SCRE) of the order to close the hatches. At 1100 hours, the midnight shift maintenance crew was sent to the torus to complete the hatch installation. The midnight shift crew was able to observe that the afternoon maintenance crew was finishing the west torus hatch and made arrangements to perform turnover when the west hatch bolting was completed. The afternoon crew completed bolting the west torus hatch, then exited the torus area.

The operations crew performed shift turnover, however the SOM did not discuss the torus hatch closure activities with his relief. Although, earlier, he did mention the torus hatch closure was in progress to his Shift Engineer. The maintenance crew performed their turnover and the oncoming crew reviewed the work package. The package did not contain instructions to notify the control room before securing the hatches. After the turnover, the oncoming crew entered the torus area and prepared the torus flange for closure and installed the torus gasket. At 0344 on 7/14/94 the MMD crew closed the torus east hatch. At 0346 the "TORUS VACUUM RELIEF VALVE NOT CLOSED" annunciator alarm actuated in the main control room. The control room operators observed that the torus vacuum breakers 3-1601-20A and 3-1601-20B and torus pressure were cycling. The vacuum breakers opened at approximately -0.46 pounds per square inch differential (psid) pressure between the reactor building and the torus. Technical Specifications section 3.7.A. require the vacuum breakers to open at less than or equal to -0.50 psid pressure between the reactor building and the torus. At the same time, the MMD crew noticed the vacuum breakers cycling and called the control room to inform the operators of the occurrence. The operator decided that the cause of the cycling was due to operation of the Drywell and Torus Purge Fan with suction from the torus. He kept the maintenance crew on the telephone and closed the torus ventilation 3-1601-60 valve. After the valve was closed, the torus to reactor building vacuum breakers stopped cycling after approximately 20 seconds.

NRC FORM 366A (5-92)	U.S. NUCLEAR R	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.						
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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

During the operations department midnight to day shift turnover, the midnight SCRE informed the oncoming Control Room Outage Engineer (CROE) of the reactor building to torus vacuum breaker operation. The oncoming CROE believed the event should be reportable and discussed the event with the oncoming Shift Engineer. After looking up the event in the Reportability Manual, the CROE decided that an Emergency Notification System (ENS) telephone call was required. The notification was made on 7/14/94 at 0739 hours.

# C. CAUSE OF EVENT:

This report is submitted in accordance with 10CFR50.73 (a)(2)(iv), which requires reporting of any event that results in automatic or manual actuation of any Engineered Safety Feature (ESF). This event occurred because the work package did not ensure the plant was in the proper configuration to perform the evolution. The SOM did not recognize the possibility of pulling a vacuum on the torus when the hatches were closed, which contributed to this event. Contributing causes of the event also include not informing the control room of the plan to close the torus hatches.

There has been one previous occurrence of an inadvertent opening of the torusto-reactor building vacuum breakers; however, the cause was a wrong component error by an instrument mechanic.

### D. SAFETY ANALYSIS:

The safety significance of this event is minimal because the reactor to torus vacuum breakers performed as designed to prevent exceeding the torus design pressure. Also the torus ventilation lineup with the torus hatches open is only done when primary containment is not required.

### E. CORRECTIVE ACTIONS:

The maintenance package used to open and close the torus hatches will be revised to notify the control room to verify ventilation lineup before closing the torus hatches due to the possibility of drawing a vacuum and cycling vacuum breaker valves. Corrective actions to prevent similar occurrences when maintenance activities are in progress are under review, and will be discussed in a supplement to this report. The event will be reviewed in licensed operator continuing training. Training will address the circumstances that resulted in drawing a vacuum in the torus, the reportability requirements when the vacuum breaker is actuated, and the corrective actions resulting from this LER.

### F. PREVIOUS OCCURRENCES:

None.

### G. COMPONENT FAILURE DATA:

None.