



**Commonwealth Edison**

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

January 17, 1994

GFS LTR: 94-0026

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

License Event Report 2-93-031, Docket 050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a)(2)(v)(D).

*[Handwritten Signature]* 1-19-94  
Gary F. Speer  
Station Manager  
Dresden Station

GFS/maf

Enclosure

cc: J. Martin, Regional Administrator, Region III  
NRC Resident Inspector's Office  
File/NRC  
File/Numerical

25000

9401280045 940117  
PDR ADOCK 05000237  
S PDR

1022  
11

LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) Dresden Nuclear Power Station, Unit 2				Docket Number (2) 0 5 0 0 0 2 3 7				Page (3) 1 of 0 4			
--	--	--	--	--------------------------------------	--	--	--	----------------------	--	--	--

Title (4)  
Reactor Vessel level Instrumentation Found Outside of Technical Specification Limits Due to Setpoint Drift

Event Date (5)			LER Number (6)					Report Date (7)			Other Facilities Involved (8)			
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)				
1	2	2 9	3 9	3 0	1 1	0 0	0 1	1 7	9 4	N/A				

OPERATING MODE (9) N  
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR  
(Check one or more of the following) (11)

POWER LEVEL (10)	20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)		
	20.405(a)(1)(i)			50.36(c)(1)			X 50.73(a)(2)(v)			73.71(c)		
	20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			Other (Specify in Abstract below and in Text)		
	20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii) (A)					
	20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii) (B)					
	20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)					

LICENSE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Mark Churilla, System Engineering	Ext. 2788
	AREA CODE: 8 1 5 9 4 2 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	A D	L I S	Y O 1 9	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

Expected Submission Date (15)	Month	Day	Year
X Yes (If yes, complete EXPECTED SUBMISSION DATE)	1	2	0 1 9 4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On December 22, 1993 at 0945 hours, with Unit 2 at 99% rated core thermal power, while performing Dresden Instrument Surveillance (DIS) 0500-3, Reactor Water Level ECCS Initiation Indicating Switch Calibration, 2-263-72C contact 5/6 was found outside of Technical Specification (TS) limits. Level Indicating Switch (LIS) 2-263-72C contact 5/6, is part of the High Pressure Coolant Injection (HPCI) and Low Pressure Coolant Injection (LPCI) Loop 1 -59 inch initiation one out of two/twice logic. LIS 2-263-72C was declared inoperable. The necessary redundant switches were available during this event allowing both systems to remain operable. The contacts were readjusted and tested satisfactorily using DIS 0500-03. The Safety Significance of this event is minimal since the redundant switches that make up the one out of two twice logic were available throughout this event. A previous event involving the failure of 2-263-72B, High Reactor Water Level HPCI Turbine Trip is documented in LER 93-019/050237.



FACILITY NAME (1)  Dresden Nuclear Power Station	DOCKET NUMBER (2)  0 5 0 0 0 2 3 7	LER NUMBER (6)						Page (3)			
		Year		Sequential Number			Revision Number				
		9 3	--	0 3	1	--	0 0	0	3	OF	0 4

TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

drift is not known at this time. A supplemental Report will be written to address the root cause failure of LIS 2-263-72C.

A history review indicated that LIS 2-263-72C was replaced during Unit 2 Refuel Outage D2R13 in 1993. Since, the LIS replacement in April 1993 the switch has experienced several setpoint drift failures.

**D. SAFETY ANALYSIS OF EVENT:**

In this event the HPCI and LPCI Systems would have initiated automatically once the reactor level reached -59 inches. The failure mode of LIS 2-263-72C was in the conservative direction thus providing a half trip condition in the one out of two twice logic prior to Vessel level reaching -59 inches. Therefore, since the redundant switches were available and the trip point of LIS 2-263-72C was in the conservative direction the safety significance is considered minimal.

**E. CORRECTIVE ACTIONS:**

The immediate corrective action readjusted and tested LIS 2-263-72C per DIS 0500-03.

The switches for the Unit 3 -59 inch initiation logic 3-263-72A, 3-263-72B, 3-263-72C and 3-263-72D will not be replaced as previously committed in PIR 3-92-116 as a result of the apparent failure increase on the Unit 2 LIS that were replaced during D2R13.

Due to the chronic failures of Yarway switches an action plan to address the setpoint drifts will be developed by 2/28/94 by the System Engineer (237-180-93-03101).

The Site Engineering Department will determine if it is feasible to replace the Yarway switches with a Rosemount Analog Trip System by 7/1/94 (237-180-93-03102).

A Technical Specification change has been submitted to change the required setpoint from 84 (+4/-0) inches of water dp to ≥84 inches of water dp (237-200-89-02203).

A review of procedure DIS 0500-03 will be performed to determine if the as found procedure steps can be enhanced by 9/1/94 (237-180-93-03103).

A supplemental report will be written by 12/1/94, providing status on the actions taken to date and results of a root cause investigation (237-180-93-03104). Further failures of the Yarway switches in 1994 that do not render the initiation logic inoperable will be summarized in this supplement. Licensed Event Reports (LERs) will be written only if a switch(es) failure would render the applicable system (s) inoperable.

**F. PREVIOUS OCCURRENCES:**

<u>LER/Docket Numbers</u>	<u>Title</u>
93019/050237	HPCI Declared Inoperable Due to Failure of the High Reactor Level Trip Switch
	The HPCI high level trip was inadvertently disabled during maintenance activities.

FACILITY NAME (1)  Dresden Nuclear Power Station	DOCKET NUMBER (2)  0   5   0   0   0   2   3   7	LER NUMBER (6)						Page (3)				
		Year		Sequential Number				Revision Number				
		9   3	--	0   3   1	--	0   0	0   4	OF	0   4			

TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

G. COMPONENT FAILURE DATA:

<u>Manufacturer</u>	<u>Nomenclature</u>	<u>Model Number</u>	<u>Mfg. Part Number</u>
Yarway	Level Switch	4418C	DS551