



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 29, 1993

Docket Nos. STN 50-456, STN 50-457,
STN 50-454, STN 50-455,
50-237, 50-249, 50-373,
50-374, 50-254, 50-265,
50-295, and 50-304

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III, Suite 500
1400 OPUS Place
Downers Grove, Illinois 60515

Dear Mr. Farrar:

SUBJECT: SAFETY EVALUATION FOR PROPOSED CHANGES TO THE EMERGENCY ACTION
LEVELS FOR COMMONWEALTH EDISON'S GENERATING STATION EMERGENCY PLAN
ANNEXES (TAC NOS. M87325, M87326, M87327, M87328, M87329, M87330,
M87331, M87332, M87333, M87334, M87335, AND M87336)

By letter dated September 1, 1993, as supplemented by letters dated October 1, 1993, November 30, 1993, December 17, December 21, and December 23, 1993, Commonwealth Edison Company (CECo) submitted proposed changes to the emergency action levels (EALs) for the Generating Station Emergency Plan (GSEP) Annexes. They were reviewed against the guidance in NUMARC/NESP-007, Rev. 2, "Methodology for Development of Emergency Action Levels." NUMARC/NESP-007 has been endorsed by the NRC in Regulatory Guide 1.101, "Emergency Planning and Preparedness for Nuclear Reactors," as an alternative means by which licensees can meet the requirements of 10 CFR 50.47(b)(4) and Appendix E to 10 CFR Part 50.

We have completed our review of the submittals and concluded that with the exception of the departures from the NUMARC guidance that were identified and accepted, the revised EALs are consistent with the guidance in NUMARC/NESP-007, and therefore, meet the requirements of 10 CFR 50.47(b)(4) and Appendix E to 10 CFR Part 50.

With one exception, all EALs will be implemented by January 1, 1994. The exception is the inclusion of an EAL for evaluation of the Fuel Clad and Fission Product Barrier Matrix which will be implemented by March 23, 1994. For the Potential Loss of the Fuel Clad Barrier condition, CECo committed to add an additional Threshold Value to account for the condition when the

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Mr. D. L. Farrar

-2-

average of the ten highest core exit thermocouples indicate 708°F for Byron and Braidwood Stations, and 700°F for Zion Station.

Our Safety Evaluations are enclosed.

Sincerely,

Original Signed By:

George F. Dick, Jr., Senior Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
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