



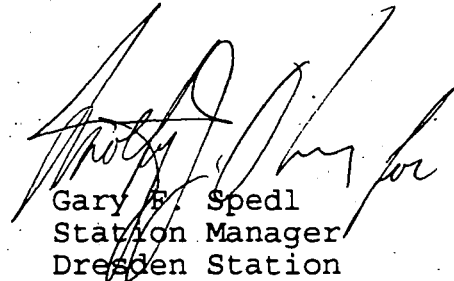
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Dresden Nuclear Power Station
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October 25, 1993

GFSLTR 93-0108

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

License Event Report 93-019, Docket 050237 is being submitted as
required by Technical Specification 6.6. NUREG 1022 and 10 CFR
50.73 (a) (2) (v) (D)


Gary E. Spedl
Station Manager
Dresden Station

10-25-93

GFS:slb

enclosure

cc: J. Martin, Regional Administrator, Region III
NRC Resident Inspector's Office
File/NRC
File/Numerical

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LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) Dresden Nuclear Power Station, Unit 2										Docket Number (2) 0 5 0 0 0 2 3 7					Page (3) 1 of 0 3								
Title (4) High Pressure Coolant Injection Declared Inoperable Due to Failed High Reactor Water Level Turbine Trip Switch																							
Event Date (5)			LER Number (6)					Report Date (7)			Other Facilities Involved (8)												
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)													
1	0	0	1 9 3	9	3	—	0	1	9	—	0	0	1	0	2	9	9	3	N/A				
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)																					
POWER LEVEL (10)		0		9		9		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)									
								20.405(a)(1)(i)		50.36(c)(1)		X 50.73(a)(2)(v)		73.71(c)									
								20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		Other (Specify in Abstract below and in Text)									
								20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii) (A)											
								20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii) (B)											
								20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)											
LICENSE CONTACT FOR THIS LER (12)																							
NAME Mark Churilla, System Engineering										TELEPHONE NUMBER Ext. 2788													
AREA CODE 8 1 5					9 4 2 2 9 2 0																		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS														
X	B J	L I S Y	0 1 0	Y																			
SUPPLEMENTAL REPORT EXPECTED (14)										Expected Submission Date (15)		Month	Day	Year									
Yes (if yes, complete EXPECTED SUBMISSION DATE)										X	NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 1, 1993 at 1130 hours, with Unit 2 at 99% rated core thermal power, while performing Dresden Instrument Surveillance (DIS) 500-3, Reactor Water Level ECCS Initiation Indicating Switch Calibration, level switch 2-263-72B contact 3/4, High Pressure Coolant Injection (HPCI) Turbine Trip, failed to operate. The HPCI System was declared inoperable and a seven day Limiting Condition for Operation (LCO) was entered per Technical Specification (TS) 3.5. The Instrument Maintenance Department (IMD) replaced the High Reactor Water Level HPCI Turbine Trip Switch. The switch was calibrated and DIS 0500-3 was performed satisfactorily. The seven day LCO was terminated on October 2, 1993. The Safety Significance of this event was minimal since all other Emergency Core Cooling Systems (ECCS) required by TS 3.5.C.2.a were operable. A previous event involving the failure of the High Reactor Water Level HPCI Turbine Trip is documented in LER 93-018/050249.

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7	LER NUMBER (6)						Page (3)		
		Year		Sequential Number		Revision Number				
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TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

trip on high reactor water level to operable status while repairs were made. The mercury switch was replaced and the magnet was readjusted. DIS 0500-3 was performed satisfactorily and the seven day LCO was terminated on October 2, 1993 at 0001 hours.

A history review indicated a similar event occurred on Unit 3 in August of 1993. During the investigation into that event it was determined that the HPCI High Reactor Water Level Trip was disabled. The Low Low Reactor Water Level (-59 inches) ECCS initiation switch contacts were found out of tolerance on LIS 3-263-72B. In order to provide a half initiation signal on the -59 inch contacts, the entire level switch was valved out, with the indication at the full downscale position. With the level switch in the downscale position, the HPCI High Reactor Water Level Turbine Trip was effectively disabled.

D. SAFETY ANALYSIS OF EVENT:

The HPCI system is designed to automatically cycle between Reactor Low Low Level (-59 inches) and Reactor High Level (+48 inches).

In this event the HPCI Turbine would not trip automatically once reactor level reaches +48 inches. However, the remote turbine trip (located in the main control room) was functional and was available to trip the HPCI turbine on high reactor water level. Therefore, since manual action could of been taken to trip the HPCI Turbine on high reactor water level and all ECCs required by the T.S. 3.5.C.2.a were operable throughout this event, the safety significance was minimal.

E. CORRECTIVE ACTIONS:

The immediate corrective actions had the IMD installed a jumper to provide a half trip signal condition for the HPCI system. The IMD completed repairs under Nuclear Work Request D22095. DIS 0500-3 was performed satisfactorily and the seven day LCO was terminated.

Due to the continuing problems with the Yarway level indicating switches, the Station has proceeded to investigate possible alternatives to the Yarway switches. Engineering review of cost estimates is in progress (237-200-91-111901).

F. PREVIOUS OCCURRENCES:

<u>LER/Docket Numbers</u>	<u>Title</u>
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93-018/050249	HPCI Outside of FSAR Design Requirements Due to Disabled Turbine Trip
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The HPCI High Reactor Water Level Turbine Trip was disabled in order to maintain a half initiation condition on the -59 inch initiation contacts.

G. COMPONENT FAILURE DATA:

<u>Manufacturer</u>	<u>Nomenclature</u>	<u>Model Number</u>	<u>Mfg. Part Number</u>
Yarway	Level Indicating Switch	4418C	DS551