



Commonwealth Edison
Dresden Nuclear Power Station
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September 01, 1993

GFSLTR 93-0060

U.S. Nuclear Regulatory Commission
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Washington, D.C. 20555

License Event Report 93-013, Docket 050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a) (2) (v).

Alt. Action for 9293

Gary F. Spedl
Station Manager
Dresden Station

GFS/slb

Enclosure

cc: Mr. J. Martin, Regional Administrator, Region III
NRC Resident Inspector's Office
File/NRC
File/Numerical

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LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) Dresden Nuclear Power Station, Unit 3	Docket Number (2) 0 5 0 0 0 2 4 9	Page (3) 1 of 0 3
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Title (4)
High Pressure Coolant Injection Declared Inoperable Due to Control Valve Limit Switch Failure

Event Date (5)			LER Number (6)				Report Date (7)			Other Facilities Involved (8)			
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)			
0	8	0	9	9	3	0	1	3	N/A				
0	8	0	9	9	3	0	1	3	N/A				

OPERATING MODE (9) N THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

POWER LEVEL (10)	1	0	0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
				20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
				20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	Other (Specify in Abstract below and in Text)
				20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii) (A)	
				20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii) (B)	
			20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)		

LICENSE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Mark Churilla, System Engineer	Ext. 2788
	8 1 5 9 4 2 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	B J	3 3	N O I 5	Y					

SUPPLEMENTAL REPORT EXPECTED (14)				Expected Submission Date (15)	Month	Day	Year
Yes (If yes, complete EXPECTED SUBMISSION DATE)				X NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 9, 1993 at 1300 hours, with Unit 3 at 100% rated core thermal power, while performing Dresden Operating Surveillance (DOS) 2300-3, High Pressure Coolant Injection (HPCI) Monthly Surveillance Test, the HPCI Turbine Failed to reset following a trip test. The HPCI System was declared inoperable and a seven day Limiting Condition for Operation (LCO) was entered per Technical Specification (TS) 3.5. A work request was written for the Electrical Maintenance Department (EMD) to inspect the Turbine Trip Logic. It was determined that Limit Switch (LS) 7, Control Valve position, was not making contact with the control valve plunger which prevented the Turbine from being reset from the control room. The EMD readjusted LS7 and the HPCI Turbine was tripped and reset satisfactorily several times. DOS 2300-3 was performed satisfactorily and the seven day LCO was terminated on August 10, 1993. The Safety Significance of this event is minimal since all other Emergency Core Cooling Systems (ECCS) required by TS 3.5.C.2.a were operable. There have been no similar events involving the inability to reset the HPCI Turbine.

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TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

PLANT AND SYSTEM IDENTIFICATION:

General Electric-Boiling Water Reactor-2527 Mwt rated core thermal power.

Nuclear Tracking System (NTS) tracking code numbers are identified in the text as (XXX-XXX-XX-XXXXX)

EVENT IDENTIFICATION:

High Pressure Coolant Injection Declared Inoperable Due to Control Valve Limit Switch Failure

A. CONDITIONS PRIOR TO EVENT:

Unit: 3 Event Date: August 9, 1993 Event Time: 1300
 Reactor Mode: N Mode Name: Run Power Level: 100
 Reactor Coolant System (RCS) Pressure: 1002 PSIG

B. DESCRIPTION OF EVENT:

On August 9, 1993 at 1300 hours, with Unit 3 at 100% rated core thermal power, while performing Dresden Operating Surveillance (DOS) 2300-3, High Pressure Coolant Injection (HPCI) Monthly Operability Test, the HPCI Turbine failed to reset from the control room following a trip test. Without the ability to reset the turbine from the control room the HPCI system was declared inoperable and a seven day Limiting Condition for Operation (LCO) was entered per Technical Specification (TS) 3.5. A work request was written for the Electrical Maintenance Department (EMD) to inspect the turbine trip logic. It was determined that Limit Switch (LS) 7, Control Valve position, was not making contact with the Control Valve plunger. The EMD readjusted LS7 and verified the turbine was resettable. DOS 2300-3 was performed satisfactorily, and the seven day LCO was terminated on August 10, 1993. The Safety Significance of this event is minimal since all other Emergency Core Cooling Systems (ECCS) required by TS 3.5.C.2.a were operable. There have been no other events involving the failure of the HPCI Turbine to reset.

C. APPARENT CAUSE OF EVENT:

This report is being submitted in accordance with 10CFR50.73 (a)(2)(v)(D), which requires the reporting of any condition that could have prevented the fulfillment of a safety system needed to mitigate the consequences of an accident.

The HPCI Turbine can be reset if the Control Valves are fully closed (LS7), Hydraulic Oil pressure is greater than 65 psig (PS1), and no turbine trips exists. Work Request 21074 was written to troubleshoot the aforementioned items. It was discovered that LS7, Control Valve Position, was not making contact with Control Valve plunger. This resulted in an open in the HPCI Turbine trip reset logic. It is believed that LS7 had become loose during previous HPCI System testing. The limit switch arm which was found loose was forced out of adjustment by the Control Valves plunger's downward force following the trip test. The arm and switch was examined and found to be acceptable. LS7 was readjusted and the turbine was tripped and reset several times. DOS 2300-3 was satisfactorily performed and the seven day LCO was terminated on August 10, 1993.

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TEXT Energy Industry Identification System (EIS) codes are identified in the text as [XX]

D. SAFETY ANALYSIS OF EVENT:

In this event the Turbine could not be reset remotely during DOS 2300-3 Turbine trip test. In this condition the HPCI System was not able to automatically initiate. However, since the failure of LS7 was caused by the monthly surveillance test the HPCI system was operable prior to this event. Therefore, since the HPCI System was operable prior to this event and the Isolation Condenser and Automatic Depressurization systems were operable throughout this event, the safety significance is minimal.

E. CORRECTIVE ACTIONS:

The EMD readjusted LS7 and successfully tripped and reset the turbine from the control room under Work Request 21074.

The System Engineer verified that LS7 on Unit 2 was tight and adjusted properly during D2R13 startup testing following the Turbine Overhaul in May of 1993.

The EMD will determine if a set screw can be installed on the Unit 2 and Unit 3 wiper arm for LS7 by March 12, 1994 (249-180-93-01301).

F. PREVIOUS OCCURRENCES:

None

G. COMPONENT FAILURE DATA:

<u>Manufacturer</u>	<u>Nomenclature</u>	<u>Model Number</u>	<u>Mfg. Part Number</u>
National Acme	Limit Switch	D2400X-J1	D2400X-J1