



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

February 16, 1993

CWS PMLTR #: 93-0088

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #93-003-0, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a)(2)(i)(A).

Charles W. Schroeder for 2-16-93

Charles W. Schroeder
Station Manager
Dresden Station

CWS/slb

Enclosure

cc: A. Bert Davis, Regional Administrator, Region III
File/NRC
File/Numerical

220108

(g:\wpt\cws1t93\93-0088)

9302230291 930218
PDR ADOCK 05000237
S PDR

JE22

LICENSEE EVENT REPORT (LER)

Form Rev 2.0

Facility Name (1) Dresden Nuclear Power Station, Unit 2 Docket Number (2) 0 5 10 10 10 2 3 17 Page (3) 1 of 0 3

Title (4) Outboard Main Steam Line Isolation Valve 2-203-2A As Found Leakage Rate Exceeded the Technical Specification Limit of 11.5 scfh

Event Date (5)			LER Number (6)				Report Date (7)			Other Facilities Involved (8)													
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)													
0	1	9	9	3	0	0	2	1	8	9	3	0	5	10	10	10	2	3	17	1	of	0	3

OPERATING MODE (9) N

POWER LEVEL (10) 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> Other (Specify in Abstract below and in Text)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

Name: M. Andjelic, LLRT Coordinator Telephone Number: 8 1 5 9 4 2 -2 19 2 10

Ext. 2366

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	X	B	I S V C	C 6 6 5	Y				

SUPPLEMENTAL REPORT EXPECTED (14)

Expected Submission Date (15) 0 7 2 1 9 3

X Yes (If yes, complete EXPECTED SUBMISSION DATE) NO

ABSTRACT (Limit to 1400 spaces, i.e, approximately fifteen single-space typewritten lines) (16)

On January 19, 1993 with Unit 2 in a refuel Outage, the performance of Dresden Technical Surveillance (DTS) 250-03, Main Steam Isolation Valve Local Leak Rate (Wet) Test, identified that the "A" Main Steam Line Outboard isolation Valve [SB], 2-203-2A, was leaking 14.96 scfh. This leakage rate exceeds the limit specified in Technical Specification 3.7.A.2.b.2.c, which limits the leakage past any main steam isolation valve (MSIV) to 11.5 scfh when tested with air at a pressure of 25 psig. The Shift Engineer was notified that the leakage rate for the 2-203-2A MSIV had exceeded the Technical Specification limit of 11.5 scfh and a Problem Identification Form (PIF) was initiated per Dresden Administrative Procedure (DAP) 02-27, Integrated Reporting Process. Work Request (WR) 15596 was written to inspect and repair the valve to reduce leakage. The safety significance of the leakage past outboard MSIV 2-203-2A has been considered to be minimal since the redundant inboard MSIV 2-203-1A had a leakage rate of 2.39 scfh; therefore, the total through leakage out of the penetration, on a minimum pathway basis, was 2.39 scfh and would not cause the maximum off site dose rates established in 10 CFR 100 to be exceeded. MSIV 2-203-2A will be inspected and repaired to identify the cause of the unsatisfactory leakage. In addition, inboard MSIVs 2-203-1C and 2-203-1D will be repaired under work requests 10044 and 15715, since the as-found leakage past these valves indicate an upward trend. A supplement to this report will be submitted by July 21, 1993 to report the cause of failure, corrective actions, and final leakage results for MSIVs 2-203-2A, 1C, and 1D (237-180-93-00301).

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Form Rev 2.0

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7	LER NUMBER (6)			Page (3)		
		Year 9 3	Sequential Number - 0 0 3	Revision Number - 0 0			
TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]							

PLANT AND SYSTEM IDENTIFICATION:

General Electric - Boiling Water Reactor -2527 Mwt rated core thermal power.

Nuclear Tracking System (NTS) tracking code numbers are identified in the text as (XXX-XXX-XX-XXXXX).

EVENT IDENTIFICATION:

Outboard Main Steam Line Isolation Valve 2-203-2A As Found Leakage Rate Exceeded the Technical Specification Limit of 11.5 scfh.

A. CONDITIONS PRIOR TO EVENT:

Unit: 2 Event Date: January 19, 1993 Event Time: 1300 hrs
 Reactor Mode: N Mode Name: Refuel Power Level: 0%

B. DESCRIPTION OF EVENT:

On January 19, 1993 with Unit 2 in a refuel Outage, the performance of Dresden Technical Surveillance (DTS) 250-03, Main Steam Isolation Valve Local Leak Rate (Wet) Test, identified that the "A" Main Steam Line Outboard isolation Valve [SB], 2-203-2A, was leaking 14.96 scfh. This leakage rate exceeds the limit specified in Technical Specification 3.7.A.2.b.2.c, which limits the leakage past any main steam isolation valve (MSIV) to 11.5 scfh when tested with air at a pressure of 25 psig. DTS 250-03 challenges the integrity of the outboard MSIVs and was performed as a result of obtaining a leakage rate of 17.35 scfh when performing DTS 250-01, Main Steam Isolation Valve Local Leak Rate (DRY) Test. DTS 250-01 challenges the integrity of both the inboard and outboard MSIVs; therefore, DTS 250-03 must be performed on any set of MSIVs which leak in excess of 11.5 scfh, when tested per DTS 250-01, to quantify the leakage past the outboard MSIV.

The Shift Engineer was notified that the leakage rate for the 2-203-2A MSIV had exceeded the Technical Specification limit of 11.5 scfh, and a Problem Identification Form (PIF) was initiated per Dresden Administrative Procedure (DAP) 02-27, Integrated Reporting Process. Work Request (WR) 15596 was written to inspect and repair the valve to reduce leakage.

C. APPARENT CAUSE OF EVENT:

This report is being submitted in accordance with 10 CFR 50.73(a)(2)(i), which requires the reporting of any operation or condition prohibited by the Technical Specifications.

The cause of the unsatisfactory leakage past MSIV 2-203-2A is currently under investigation. This valve will be repaired and tested in accordance with DTS 250-01 or DTS 250-03 prior to the end of D2R13 Refuel Outage. A supplement to this report will be submitted by July 21, 1993 to report the cause of failure, corrective actions, and final leakage results (237-180-93-00301).

D. SAFETY ANALYSIS OF EVENT:

The safety significance of the leakage past outboard MSIV 2-203-2A has been considered to be minimal, since the redundant inboard MSIV 2-203-1A had a leakage rate of 2.39 scfh; therefore, the total through leakage out of the penetration, on a minimum pathway basis, was 2.39 scfh and would not cause the maximum off site dose rates established in 10 CFR 100 to be exceeded.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Form Rev 2.0

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 3 7	LER NUMBER (6)			Page (3)		
		Year 9 3	Sequential Number - 0 0 3	Revision Number - 0 0			
TEXT Energy Industry Identification System (EIIS) codes are identified in the text as [XX]							

E. CORRECTIVE ACTIONS:

MSIV 2-203-2A will be inspected and repaired to identify the cause of the unsatisfactory leakage. In addition, inboard MSIVs 2-203-1C and 2-203-1D will be repaired under work requests 10044 and 15715 since the as found leakage past these valves indicate an upward trend. MSIV 203-1C had a leakage rate of 4.50 scfh at the beginning of fuel cycle 13. The as found testing of MSIV 2-203-1C at the end of fuel cycle 13 indicated a leakage rate of 7.86 scfh. Likewise, MSIV 2-203-1D had a leakage rate of 3.70 scfh at the beginning of fuel cycle 13. The as found testing of MSIV 2-203-1C at the end of fuel cycle 13 indicated a leakage rate of 7.86 scfh. A supplement to this report will be submitted by July 21, 1993 to report the cause of failure, corrective actions, and final leakage results for MSIVs 2-203-2A, 1C, and 1D (237-180-93-00301).

F. PREVIOUS OCCURRENCES:

<u>LER/Docket Numbers</u>	<u>Title</u>
90-009/0500237	Type B and c Primary Containment Local Leak Rate Test Requirements Exceeded Due To Leaking Isolation Valves

G. COMPONENT FAILURE DATA:

<u>Manufacturer</u>	<u>Nomenclature</u>	<u>Model Number</u>	<u>Mfg. Part Number</u>
Crane Co.	2-203-2A	DR34289-20" Y Pattern Globe Valve	N/A

An Industry-wide data base search will be performed and included in the supplemental report (237-180-93-00301).