

OCT 27 1992

ENCLOSURE 7

MEMORANDUM FOR: John A. Zwolinski, Assistant Director  
for Region III Reactors  
Division of Reactor Projects - III/IV/V  
Office of Nuclear Reactor Regulation

FROM: Hubert J. Miller, Director  
Division of Reactor Safety, Region III

SUBJECT: REQUEST FOR TECHNICAL ASSISTANCE  
INTERPRETATION AND RESOLUTION OF  
REGULATORY GUIDE (RG) 1.97 AT QUAD CITIES,  
DOCKET NUMBERS 50-254 AND 50-265  
(AITS 92-0641)

Determine if the licensee complies with RG 1.97, Revision 2 for the neutron flux monitoring instrumentation, the reactor level instruments, and the Moore Industries signal conditioners.

Four unresolved items remain open.

Unresolved item (254/88027-01(DRS);265/88028-01(DRS)): Determine adequacy of the neutron flux monitoring instrumentation in meeting the Category 1 requirements. The licensee's position is to follow the BWR Owner's Group recommendations as stated during subsequent discussions with the NRC, which indicated that further review by NRR was required.

Unresolved item (254/88027-03(DRS);265/88028-03(DRS)): Evaluate and determine acceptability of the licensee analysis concerning the off-scale indication of the reactor level indicators 263-73A and 263-73B with the recirculation pumps running as identified in the licensee's letter dated December 9, 1991. Based upon the added instrumentation and discussions with NRR/SICB, further review and analysis by NRR is required.

Unresolved item (254/88027-05(DRS);265/88028-05(DRS)): Evaluate and determine acceptance of the licensee's use of Moore Industries signal conditioners as isolators. Maximum credible fault testing of these devices was completed per Test Report CWE-3212P, Revision 2, dated November 4, 1988, copies of which were previously forwarded to NRR/SICB (Model SCT/0-1V/0-1V/24Vdc(STD)).

Unresolved item (254/88027-06(DRS);265/88028-06(DRS)): Evaluate and determine acceptance of the licensee's use of Moore Industries signal conditioners as isolators. Maximum credible fault testing of these devices was completed per Test Report CWE-3480, Revision 0, dated October 12, 1989, copies of which were previously forwarded to NRR/SICB ((Models SCT/4-20mA/4-20mA/117Vac(STD) and MVT/80-160mV/4-20mA/117Vac(STD)).

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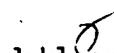
Direct any questions concerning this subject to either George Hausman at (708) 790-5523 or Frank Jablonski at (708) 790-5555. We consider this to be a Priority 2 item with a completion date of January 16, 1993.

ORIGINAL SIGNED BY HUBERT J. MILLER

Hubert J. Miller, Director  
Division of Reactor Safety

Attachment: Commonwealth  
Edison Letter dated  
December 9, 1991

cc w/attachment  
L. N. Olshan, NRR  
A. G. Hansen, NRR  
B. S. Marcus, NRR  
S. F. Newberry, NRR  
DCD/DCB(RIDS)

RIII	RIII	RIII	RIII	RIII	RIII
					
Hausman	Jablonski	Hasse	Ring	Martin	Miller
10/26/92	10/26/92	10/26/92	10/26/92	10/27/92	10/27/92