



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

September 8, 2017

Mr. Tom Simril
Site Vice President
Catawba Nuclear Station, Units 1 and 2
Duke Energy Carolinas, LLC
4800 Concord Road
York, SC 29745

**SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2 – ISSUANCE OF
AMENDMENTS ADOPTING TECHNICAL SPECIFICATION TASK FORCE
TRAVELER TSTF 283-A, REVISION 3 (CAC NOS. MF8975 AND MF8976)**

Dear Mr. Simril:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 292 to Renewed Facility Operating License No. NPF-35 and Amendment No. 288 to Renewed Facility Operating License No. NPF-52 for the Catawba Nuclear Station, Units 1 and 2, respectively. The amendments are in response to your application dated December 15, 2016 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16350A422).

The amendments modify Technical Specification (TS) 3.8.1, "AC Sources - Operating," to allow greater flexibility in performing Surveillance Requirements (SRs) by modifying Mode restriction notes in TS SRs 3.8.1.11, 3.8.1.16, 3.8.1.17, 3.8.1.19, 3.8.4.8, and 3.8.4.9. This proposed change is consistent with Technical Specification Task Force (TSTF) Traveler TSTF-283-A, Revision 3, "Modify Section 3.8 Mode Restriction Notes."

T. Simril

- 2 -

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read 'Michael Mahoney', with a long horizontal flourish extending to the right.

Michael Mahoney, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

1. Amendment No. 292 to NPF-35
2. Amendment No. 288 to NPF-52
3. Safety Evaluation

cc w/enclosures: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 292
Renewed License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility), Renewed Facility Operating License No. NPF-35, filed by Duke Energy Carolinas, LLC (licensee), dated December 15, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

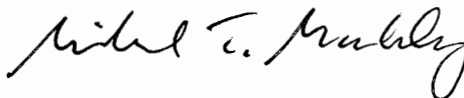
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 292 which are attached hereto, are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to License No. NPF-35
and Technical Specifications

Date of Issuance: September 8, 2017



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 288
Renewed License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility), Renewed Facility Operating License No. NPF-52, filed by the Duke Energy Carolinas, LLC (the licensee), dated December 15, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

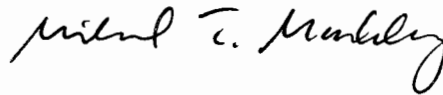
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 288, which are attached hereto, are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to License No. NPF-52
and Technical Specifications

Date of Issuance: September 8, 2017

ATTACHMENT

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

LICENSE AMENDMENT NO. 292

RENEWED FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

LICENSE AMENDMENT NO. 288

RENEWED FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

NPF-35, page 4
NPF-52, page 4

Insert

NPF-35, page 4
NPF-52, page 4

Replace the following pages of the Appendix A Technical Specifications (TS) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

TS 3.8.1-9
TS 3.8.1-12
TS 3.8.1-13
TS 3.8.1-14
TS 3.8.4-3
TS 3.8.4-4

Insert

TS 3.8.1-9
TS 3.8.1-12
TS 3.8.1-13
TS 3.8.1-14
TS 3.8.4-3
TS 3.8.4-4

(2) TECHNICAL SPECIFICATIONS

The Technical Specifications contained in Appendix A, as revised through Amendment No. 292 which are attached hereto, are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than December 6, 2024, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

(4) Antitrust Conditions

Duke Energy Carolinas, LLC shall comply with the antitrust conditions delineated in Appendix C to this renewed operating license.

(5) Fire Protection Program

Duke Energy Carolinas, LLC shall implement and maintain in effect all provisions of the approved fire protection program that complies with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the licensee amendment request dated September 25, 2013; as supplemented by letters dated January 13, 2015; January 28, 2015; February 27, 2015; March 30, 2015; April 28, 2015; July 15, 2015; August 14, 2015; September 3, 2015; December 11, 2015; January 7, 2016; March 23, 2016; June 15, 2016; August 2, 2016; September 7, 2016; and January 26, 2017, as approved in the SE dated February 8, 2017. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would require prior NRC approval, the licensee may make changes to the fire protection program without prior approval of the Commission if those changes satisfy the provisions set forth in 10 CFR 50.48(a) and 10 CFR 50.48(c), the change does not require a change to a technical specification or a license condition, and the criteria listed below are satisfied.

(2) TECHNICAL SPECIFICATIONS

The Technical Specifications contained in Appendix A, as revised through Amendment No. 288, which are attached hereto, are hereby incorporated into this renewed operating license. Duke Energy Carolinas, LLC shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than December 6, 2024, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section

(4) Antitrust Conditions

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Duke Energy Carolinas, LLC shall implement and maintain in effect all provisions of the approved fire protection program that complies with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the licensee amendment request dated September 25, 2013, as supplemented by letters dated January 13, 2015; January 28, 2015; February 27, 2015; March 30, 2015; April 28, 2015; July 15, 2015; August 14, 2015; September 3, 2015; December 11, 2015; January 7, 2016; March 23, 2016; June 15, 2016; August 2, 2016; September 7, 2016; and, January 26, 2017, as approved in the SE dated February 8, 2017. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would require prior NRC approval, the licensee may make changes to the fire protection program without prior approval of the Commission if those changes satisfy the provisions set forth in 10 CFR 50.48(a) and 10 CFR 50.48(c), the change does not require a change to a technical specification or a license condition, and the criteria listed below are satisfied.

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.11 -----NOTES-----</p> <ol style="list-style-type: none"> 1. All DG starts may be preceded by an engine prelube period. 2. This Surveillance shall not normally be performed in MODE 1, 2, 3, or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR. <p>-----</p> <p>Verify on an actual or simulated loss of offsite power signal:</p> <ol style="list-style-type: none"> a. De-energization of emergency buses; b. Load shedding from emergency buses; c. DG auto-starts from standby condition and: <ol style="list-style-type: none"> 1. energizes the emergency bus in ≤ 11 seconds, 2. energizes auto-connected shutdown loads through automatic load sequencer, 3. maintains steady state voltage ≥ 3950 V and ≤ 4580 V, 4. maintains steady state frequency ≥ 58.8 Hz and ≤ 61.2 Hz, and 5. supplies auto-connected shutdown loads for ≥ 5 minutes. 	<p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.15 -----NOTES-----</p> <ol style="list-style-type: none"> 1. This Surveillance shall be performed within 5 minutes of shutting down the DG after the DG has operated ≥ 1 hour loaded ≥ 5600 kW and ≤ 5750 kW or until operating temperature is stabilized. <p style="margin-left: 40px;">Momentary transients outside of load range do not invalidate this test.</p> <ol style="list-style-type: none"> 2. All DG starts may be preceded by an engine prelube period. <p>-----</p> <p>Verify each DG starts and achieves, in ≤ 11 seconds, voltage ≥ 3950 V, and frequency ≥ 57 Hz and maintains steady state voltage ≥ 3950 V and ≤ 4580 V and frequency ≥ 58.8 Hz and ≤ 61.2 Hz.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.8.1.16 -----NOTE-----</p> <p>This Surveillance shall not normally be performed in MODE 1, 2, 3, or 4. However, this Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR.</p> <p>-----</p> <p>Verify each DG:</p> <ol style="list-style-type: none"> a. Synchronizes with offsite power source while loaded with emergency loads upon a simulated restoration of offsite power; b. Transfers loads to offsite power source; and c. Returns to standby operation. 	<p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.17 -----NOTE----- This Surveillance shall not normally be performed in MODE 1, 2, 3, or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR. -----</p> <p>Verify, with a DG operating in test mode and connected to its bus, an actual or simulated ESF actuation signal overrides the test mode by:</p> <ul style="list-style-type: none"> a. Returning DG to standby operation; and b. Automatically energizing the emergency load from offsite power. 	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.8.1.18 Verify interval between each sequenced load block is within the design interval for each automatic load sequencer.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.1.19 -----NOTES-----</p> <ol style="list-style-type: none"> 1. All DG starts may be preceded by an engine prelube period. 2. This Surveillance shall not normally be performed in MODE 1, 2, 3, or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR. <p>-----</p> <p>Verify on an actual or simulated loss of offsite power signal in conjunction with an actual or simulated ESF actuation signal:</p> <ol style="list-style-type: none"> a. De-energization of emergency buses; b. Load shedding from emergency buses; and c. DG auto-starts from standby condition and: <ol style="list-style-type: none"> 1. energizes the emergency bus in ≤ 11 seconds, 2. energizes auto-connected emergency loads through load sequencer, 3. achieves steady state voltage ≥ 3950 V and ≤ 4580 V, 4. achieves steady state frequency ≥ 58.8 Hz and ≤ 61.2 Hz, and 5. supplies auto-connected emergency loads for ≥ 5 minutes. 	<p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.8.4.4 Verify DC channel and DG battery cells, cell plates, and racks show no visual indication of physical damage or abnormal deterioration that could degrade battery performance.	In accordance with the Surveillance Frequency Control Program
SR 3.8.4.5 Remove visible terminal corrosion, verify DC channel and DG battery cell to cell and terminal connections are clean and tight, and are coated with anti-corrosion material.	In accordance with the Surveillance Frequency Control Program
SR 3.8.4.6 Verify all DC channel and DG battery connection resistance values meet Table 3.8.4-1 limits.	In accordance with the Surveillance Frequency Control Program
SR 3.8.4.7 Verify each DC channel battery charger supplies ≥ 200 amps and the DG battery charger supplies ≥ 75 amps with each charger at ≥ 125 V for ≥ 8 hours.	In accordance with the Surveillance Frequency Control Program
SR 3.8.4.8 -----NOTES----- 1. The modified performance discharge test in SR 3.8.4.9 may be performed in lieu of the service test in SR 3.8.4.8. 2. This Surveillance shall not normally be performed for the DG batteries in MODE 1, 2, 3, or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR. ----- Verify DC channel and DG battery capacity is adequate to supply, and maintain in OPERABLE status, the required emergency loads for the design duty cycle when subjected to a battery service test.	In accordance with the Surveillance Frequency Control Program

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.8.4.9 -----NOTE----- This Surveillance shall not normally be performed for the DG batteries in MODE 1, 2, 3, or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR. -----</p> <p>Verify DC channel and DG battery capacity is \geq 80% of the manufacturer's rating when subjected to a performance discharge test or a modified performance discharge test.</p>	<p>In accordance with the Surveillance Frequency Control Program</p> <p><u>AND</u></p> <p>18 months when battery shows degradation or has reached 85% of expected life with capacity < 100% of manufacturer's rating</p> <p><u>AND</u></p> <p>-----NOTE----- Not applicable to DG batteries -----</p> <p>24 months when battery has reached 85% of the expected life with capacity \geq 100% of manufacturer's rating</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 292 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-35

AND

AMENDMENT NO. 288 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-52

DUKE ENERGY CAROLINAS, LLC

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated December 15, 2016 (Agencywide Documents and Access Management System (ADAMS) Accession No. ML16350A422), Duke Energy Carolinas, LLC (the licensee), submitted a license amendment request (LAR) for the Catawba Nuclear Station (CNS), Units 1 and 2, to adopt multiple Technical Specification Task Force (TSTF) Travelers. This safety evaluation specifically evaluates the adoption of TSTF-283-A, Revision 3, "Modify Section 3.8 Mode Restriction Notes," for CNS.

TSTF Traveler 283-A, Revision 3, "Modify Section 3.8 Mode Restriction Notes," introduced a modification of the mode restriction note at the top of 14 different surveillance requirements (SRs) in the NUREG-1431, "Standard Technical Specifications: Westinghouse Plants." The licensee has chosen to adopt six of the modifications.

Specifically, CNS Technical Specification (TS) 3.8.1, "AC Sources Operating," and 3.8.4, "DC Sources Operating," SRs are modified by changing the mode restriction note to allow portions of the SRs to be performed to reestablish operability provided an assessment determines the safety of the plant is maintained or enhanced. Additionally, each SR is modified by adding a sentence that allows credit to be taken for unplanned events that satisfy the SR.

2.0 REGULATORY EVALUATION

2.1 Applicable Regulations and Guidance

In Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36, the Commission established its regulatory requirements related to the content of TS. Pursuant to 10 CFR 50.36(c), TS are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements

(SRs); (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in a plant's TS.

As discussed in 10 CFR 50.36(c)(3), SRs are requirements relating to testing, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met.

NUREG-1431 contains the standardized technical specifications for the Westinghouse fleet. NUREG-1431, September 1992 (ADAMS Accession No. ML13196A330), was developed based on the criteria in the Final Commission Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors, published July 22, 1993 (58 FR 39132), and these criteria were subsequently codified by changes to 10 CFR 50.36 (60 FR 36953).

2.2 Proposed Changes

CNS SRs 3.8.1.11, 3.8.1.16, 3.8.1.17, 3.8.1.19, 3.8.4.8 and 3.8.4.9 currently limit performance of the SR to plant operating modes other than Modes 1, 2, 3, or 4 by a note placed atop each SR. The adoption of TSTF-283-A, Revision 3, will revise the SR notes to state that while the SRs are normally prohibited "in Modes 1, 2, 3, or 4," they may be performed to reestablish operability provided an assessment determines the safety of the plant is maintained or enhanced. The NRC staff approved the changes in TSTF-283-A, Revision 3, by incorporating them into Revision 2 of NUREG-1431, "Standard Technical Specifications: Westinghouse Plants," April 2001 (ADAMS Accession No. ML011090393).

In addition to the above changes in TSTF-283-A, Revision 3, the licensee proposed to modify each SR by adding a sentence to clarify that credit may be taken for unplanned events that satisfy the SR. These changes were also included in all versions of NUREG-1431, "Standard Technical Specifications: Westinghouse Plants," including the current revision, Revision 4, April 2012 (ADAMS Accession No. ML12100A222).

The licensee noted that CNS SR 3.8.4.8 is equivalent to SR 3.8.4.7 of NUREG-1431, Revision 4, and CNS SR 3.8.4.9 is equivalent to SR 3.8.4.8 in NUREG-1431, Revision 4, in the markup included in TSTF-283-A, Revision 3.

Current CNS SR 3.8.1.11 Note 2 states:

This Surveillance shall not be performed in MODE 1, 2, 3 or 4.

Revised CNS SR 3.8.1.11 Note 2 would state:

This Surveillance shall not normally be performed in MODE 1, 2, 3 or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR.

Current CNS SR 3.8.1.16 Note states:

This Surveillance shall not be performed in MODE 1, 2, 3 or 4.

Revised CNS SR 3.8.1.16 Note would state:

This Surveillance shall not normally be performed in MODE 1, 2, 3 or 4. However, this Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR.

Current CNS SR 3.8.1.17 Note states:

This Surveillance shall not be performed in MODE 1, 2, 3 or 4.

Revised CNS SR 3.8.1.17 Note would state:

This Surveillance shall not normally be performed in MODE 1, 2, 3 or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR.

Current CNS SR 3.8.1.19 Note 2 states:

This Surveillance shall not be performed in MODE 1, 2, 3 or 4.

Revised CNS SR 3.8.1.19 Note 2 would state:

This Surveillance shall not normally be performed in MODE 1, 2, 3 or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR.

Current CNS SR 3.8.4.8 Note 2 states:

This Surveillance shall not be performed for the DG [diesel generator] batteries in MODE 1, 2, 3 or 4.

Revised CNS SR 3.8.4.8 Note 2 would state:

This Surveillance shall not normally be performed for the DG batteries in MODE 1, 2, 3 or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR.

Current CNS SR 3.8.4.9 Note states:

This Surveillance shall not be performed for the DG batteries in MODE 1, 2, 3 or 4.

Revised CNS SR 3.8.4.9 Note would state:

This Surveillance shall not normally be performed for the DG batteries in MODE 1, 2, 3 or 4. However, portions of the Surveillance may be performed to reestablish OPERABILITY provided an assessment determines the safety of the plant is maintained or enhanced. Credit may be taken for unplanned events that satisfy this SR.

The licensee did not propose changes to CNS SRs 3.8.1-8, 3.8.1-9, 3.8.1-10, 3.8.1-12, 3.8.1-13, 3.8.1-14, 3.8.1-18, and 3.8.4-7 (NUREG-1431, Revision 4, SR 3.8.4-6) even though they were included as changes in TSTF-283-A, Revision 3. The licensee stated that these SRs do not include Notes restricting the MODES in which the SR may be performed, therefore, the changes that provide exceptions to the Mode restrictions are not necessary, and are not proposed to be adopted.

3.0 TECHNICAL EVALUATION

Duke Energy proposed a revision to the CNS, Units 1 and 2, SRs 3.8.1.11, 3.8.1.16, 3.8.1.17, 3.8.1.19, 3.8.4.8, and 3.8.4.9. These SRs currently have notes prohibiting performing the surveillances in Modes 1, 2, 3, or 4. The notes are modified to state that while normally prohibited "in Modes 1, 2, 3, or 4" the surveillances may be performed to reestablish operability provided an assessment determines the safety of the plant is maintained or enhanced.

The NRC staff approved the changes in TSTF-283-A, Revision 3; by incorporating them into NUREG-1431, Revision 2, issued in April 2001. The intent of TSTF-283-A is to allow some testing of the emergency diesel generators (EDGs) and Class 1E batteries in modes not currently allowed for the purpose of maintaining or reestablishing system or component operability (e.g., post corrective maintenance testing). Testing is allowed with the provision that the licensee performs a safety assessment that determines the safety of the plant would be maintained or enhanced by conducting the operability testing before the testing begins.

The NRC staff reviewed the proposed changes to the CNS SRs 3.8.1.11, 3.8.1.16, 3.8.1.17, 3.8.1.19, 3.8.4.8, and 3.8.4.9. The NRC staff finds that the changes are identical to those contained in TSTF-283-A, Revision 3, for TSs 3.8.1.11, 3.8.1.16, 3.8.1.17, 3.8.1.19, 3.8.4.7, and 3.8.4.8. The NRC staff finds that the changes would provide flexibility in outage scheduling and reduce outage critical path time, since these EDG and Class 1E battery surveillance tests would no longer have to be performed during an outage. The changes would not result in any changes to the performance of the SR or the determination of component operability.

In addition, the proposed changes could potentially avoid a plant shutdown and the associated risk if corrective maintenance (planned or unplanned) were performed during power operation. The change will also improve the licensee's flexibility in responding to an event during shutdown when other engineered safety feature (ESF) equipment may be out of service.

Finally, the deviation to allow credit to be taken for unplanned events that satisfy the SR has been included in all final versions of NUREG-1431. If any unplanned event effectively demonstrates meeting the SR requirement, the licensee may use it to determine the operability of the associated component.

The NRC staff concludes that the requirements of 10 CFR 50.36(c)(3) will continue to be met because the revised SRs provide the appropriate surveillances to ensure the necessary quality of components is maintained and the associated LCOs will be met, ensuring the safe operation of CNS, Units 1 and 2. Therefore, the NRC staff concludes that the proposed TS changes are acceptable

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, South Carolina State official was notified of the proposed issuance of the amendments on June 26, 2017. On July 18, 2017, the NRC staff confirmed that the South Carolina State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The 10 CFR Part 50 reactor license amendments change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on this finding (82 FR 19101, April 25, 2017). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Snyder, NRR

Date: September 8, 2017

SUBJECT: CATAWBA NUCLEAR STATION, UNITS 1 AND 2 – ISSUANCE OF AMENDMENTS ADOPTING TECHNICAL SPECIFICATION TASK FORCE TRAVELER TSTF 283-A, REVISION 3 (CAC NOS. MF8975 and MF8976) DATED SEPTEMBER 8, 2017

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