



Commonwealth Edison  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

D/S

May 3, 1979

BBS LTR #79-317

James G. Keppler, Regional Director  
Directorate of Regulatory Operations - Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Revised Reportable Occurrence Report #79-1/03X-1, Docket #050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation. Cause codes 12 and 13 have been revised to reflect an operator error as the principal cause of this event.

R. B. Stephenson  
Station Superintendent  
Dresden Nuclear Power Station

BBS/lcg

Enclosure

cc: Director Inspection & Enforcement  
Director of Management Information & Program Control  
File/NRC

MAY 18 1979

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CONTROL BLOCK: 

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 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L D R S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 57 CAT 58 5

7 8 9 14 15 25 26 30 57 58

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

0 1 L 0 5 0 0 0 2 3 7 0 1 0 3 7 9 8 0 5 0 3 7 9 9

7 8 60 61 68 69 74 75 80

REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

### EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During steady state operation the torus level increased to -1 inch, due to re-

0 3 petitive starts of the HPCI turbine. See RO#79-2/01T-0. Immediate operator action

0 4 was to pump the excess torus water to the hotwell, restoring water level to -3.5"

0 5 which is within specifications. This event had minimal safety significance since

0 6 the 0.5 inch excess in water level did not significantly affect the pressure sup-

0 7 pression capability. This event has occurred previously at Dresden. (Unit 2 RO#75-51)

0 8 T.S. req's level maintained between -1.5 and -4.75" at 1# D/P.

09		SYSTEM CODE S A		CAUSE CODE A	CAUSE SUBCODE A	COMPONENT CODE Z Z Z Z Z Z						COMP. SUBCODE Z	VALVE SUBCODE Z	
7	8	9	10	11	12	13	14	15	16					
LER/RO REPORT NUMBER 17		EVENT YEAR 7 9			SEQUENTIAL REPORT NO. 0 0 1		OCCURRENCE CODE 0 3		REPORT TYPE X		REVISION NO. 1			
21	22	23	24	25	26	27	28	29	30	31	32			
ACTION TAKEN H	FUTURE ACTION G	EFFECT ON PLANT Z	SHUTDOWN METHOD Z	HOURS 0 0 0 0			ATTACHMENT SUBMITTED N		NPRD-4 FORM SUB. N		PRIME COMP. SUPPLIER Z	COMPONENT MANUFACTURER Z 9 9 9		
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47

### CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Increase in the torus water level was determined to result from operator oversight.

1 1 Personnel involved were admonished to ensure level is properly maintained. The HPCI

1 2 surveillance will be changed to include a "CAUTION" statement so the torus level will

1 3 not exceed specifications during repetitive starts of the HPCI turbine. No further

1 4 action is necessary.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	E	28	0	9	5	29	NA	NA
ACTIVITY		CONTENT		AMOUNT OF ACTIVITY		LOCATION OF RELEASE			
1	6	Z	33	Z	34	NA	NA	NA	
PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION			
1	7	0	0	0	37	Z	38	NA	
PERSONNEL INJURIES		NUMBER		DESCRIPTION					
1	8	0	0	0	40	NA			
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION					
1	9	Z	42	NA					
PUBLICITY		ISSUED		DESCRIPTION					
2	0	N	44	NA					

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NRC USE ONLY

M. Wright-

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