



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

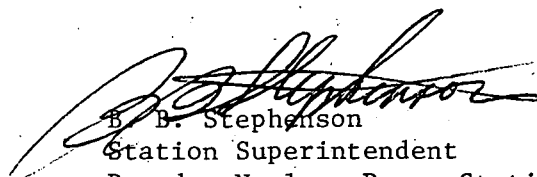
B&B

May 4, 1979

BBS LTR #79-366

James G. Keppler, Regional Director
Director of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #79-20/03L-0 Docket #050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(d), abnormal degradation of systems other than those specified in item B.1.e. above designed to contain radioactive material resulting from the fission process.


B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:lcg

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

MAY 14 1979

7905L60384

A002
5/11

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	I	L	D	R	S	2	2	0	0	-	0	0	0	0	0	0	0	0	3	4	1	1	1	1	4	5		
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34		
LICENSEE CODE														LICENSE NUMBER						LICENSE TYPE						CAT 58			

0	1	L	6	0	5	0	0	0	2	3	7	7	0	4	1	3	7	9	8	0	5	0	4	7	9	9			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34		
REPORT SOURCE												DOCKET NUMBER						EVENT DATE						REPORT DATE					

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 The measured leakage for the personnel air lock was 54.63 SCFH. The T.S.3.7 limit is

0 3 29.38 SCFH. Minimal safety significance because excessive leakage to Rx building could

0 4 only be experienced when inner airlock door is open. Similar occurrence was reported

0 5 by R.O. 50-237/71-31.

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0	9	S	A	11	E	12	B	13	P	E	N	E	T	R	14	A	15	Z	16
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26
SYSTEM CODE				CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP. SUBCODE		VALVE SUBCODE			
17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32				
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.									
A		Z		Z		Z		N		C 3 1 0									
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Leakage was identified through the outer lower handwheel shaft packing. The packing

1 1 was replaced and the personnel air lock retested satisfactorily. No additional action

1 2 is deemed necessary.

1 3

1 4

1	5	H	28	0	0	0	29	N/A	30	B	31	Local Leak Rate Test	32						
7	8	9	10	11	12	13	14	15	16	17	18	19	20						
FACILITY STATUS				% POWER				OTHER STATUS				METHOD OF DISCOVERY				DISCOVERY DESCRIPTION			
1	6	Z	33	Z	34	N/A	35	N/A	36										
ACTIVITY CONTENT				RELEASED OF RELEASE				AMOUNT OF ACTIVITY				LOCATION OF RELEASE							
1	7	0	0	0	37	Z	38	N/A	39										
PERSONNEL EXPOSURES				PERSONNEL INJURIES															
1	8	0	0	0	40	N/A	41												
LOSS OF OR DAMAGE TO FACILITY				PUBLICITY															
1	9	Z	42	N/A	43	N/A	44												
2	0	N	45	N/A	46														

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