



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

April 4, 1979

BBS LTR #79-302

James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #79-15/01T-0 Docket #050-237 is hereby submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.1.(h), errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:lbg

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

7904110 230

A002
5/11

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L R D S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 LICENSE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T

0 1 REPORT SOURCE L 0 5 0 0 0 2 3 7 7 0 3 2 1 7 9 8 0 4 0 4 7 9 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 General Electric during recent examinations of high exposure control rod blades found
0 3 significant tube cracking and some loss of B4C in control blades which had not reached
0 4 previously analyzed end-of-life exposures. The cracking occurred in a small number
0 5 of tubes as early as 80% of previous blade design exposure limit. No control rod
0 6 blades at Dresden have reached this 80% criteria at this time.
0 7

0 8
7 8 9 80

0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
7 8 9 10 11 12 13 14 15 16
R B (11) E (12) C (13) C O N R O D (14) Z (15) Z (16)

(17) LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
21 22 23 24 25 26 27 28 29 30 31 32
7 9 0 1 5 0 1 T 0

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRO-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47
G (18) C (19) Z (20) Z (21) 0 0 0 0 Y (23) N (24) N (25) G 0 8 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The mechanism of tube cracking appears to be stress corrosion cracking of stainless
1 1 steel tubing and control material swelling. During cycle 7, approximately 15 to 30
1 2 blades may reach the 80% criteria, but none should exceed the previously analyzed end
1 3 of life. Vendor recommendations for increased shutdown margin requirements or reduced
1 4 blade lifetimes will be followed.
7 8 9 80

1 5 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
H (28) 0 0 0 (29) NA D (31) Notification from General Electric

1 6 RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
Z (33) Z (34) NA NA

1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
0 0 0 (37) Z (38) NA

1 8 PERSONNEL INJURIES NUMBER DESCRIPTION (41)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
0 0 0 (40) NA

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
Z (42) NA

2 0 PUBLICITY ISSUED DESCRIPTION (45)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
N (44) NA

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NRC USE ONLY

NAME OF PREPARER

J. Doxsey

PHONE:

X-266

ATTACHMENT TO LICENSEE EVENT REPORT 79-15/01T-0
COMMONWEALTH EDISON COMPANY (CWE)
DRESDEN UNIT 2 (ILDRS 2)
DOCKET # 050-237

On March 21, 1979, General Electric informed Commonwealth Edison Company personnel that during recent examinations of two high exposure Control Rod Blades from other B.W.R. sites, significant tube cracking had occurred. There had been some loss of B4C observed in several tubes of both rods, one of which had not reached their previously analyzed end-of-life exposures. The other rod with a greater amount of cracking had slightly exceeded its end-of-life exposures. No Control Blades at Dresden have reached this 80% of end-of-life exposure.

The apparent mechanism of the tube cracking in the two inspected blades appears to be stress corrosion cracking of the stainless steel tubing, aided by B4C swelling. During the forthcoming cycle 7 on Unit 2, approximately 15 to 30 blades may reach the new criteria, but none should reach the previously analyzed end-of-life exposure. For Cycle 7, then, a G.E. analyzed shutdown margin penalty will be imposed as may be required.

The exposure of the control blades on Units 1 and 3 will not reach the new criteria prior to the next refueling outages. In these units also, vendor recommendations regarding blade lifetimes will be followed as determined necessary.