

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 21, 1995

LICENSEE: Pennsylvania Power & Light Company

FACILITY: Susquehanna Steam Electric Station, Units 1 and 2

SUBJECT: MEETING SUMMARY OF JANUARY 24, 1995

On January 24, 1995, staff from the Office of Nuclear Reactor Regulation (NRR) met with representatives from the Pennsylvania Power & Light Company (PP&L) in Rockville, Maryland. The purpose of the meeting was to discuss the contents of the November 21, 1994 PP&L amendment request to delete the main steam isolation valve leakage control system (MSIVLCS) from the Susquehanna Steam Electric Station Units 1 & 2. Enclosure 1 is a list of those who attended the meeting.

The PP&L staff provided a general discussion of the planned plant modifications, benefits to be realized, the analyses that were conducted in accordance with the BWR Owners Group (BWROG) guidance, and the schedule which is being followed (Enclosure 2). The licensee indicated that it would like to receive the license amendments by early summer to support outage planning for Unit 2. PP&L would be removing the Unit 2 LCS in the fall of 1995 and the Unit 1 LCS a year later.

The staff indicated that it has concerns about what earthquake/facility pairs PP&L had chosed to use from the BWROG data base. In addition, there was a question about what specific equipment was related to the chosen earthquake data. The staff also stated that it had some general problems with the BWROG data base but these concerns related to the basis for selected information would be pursued generically with the BWROG. Other discussion items included the scope and depth of the PP&L bounding seismic analysis, the turbine building analysis, the floor response spectra, the methodology used for the "chi over q" analyses in the Final Safety Analysis Report, and validation of in-leakage into containment.

It was agreed that the staff would complete its preliminary review of PP&L's submittal in the near term and would send to PP&L two rounds of questions to support the completion of the staff's safety evaluation. The first, to be sent within a week, will address information concerning the radiological analysis associated with the removal of the LCS. These questions had been discussed with the staff prior to and during the meeting. The second set of questions will be sent within 3 weeks of the meeting and will deal with seismic issues and the dynamic analysis completed by PP&L.

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In addition, PP&L indicated that it would be providing a supplemental submittal to address how the proposed modification complies with Appendix A to 10 CFR Part 100 and will also include a request for exemption from Appendix J to 10 CFR Part 50 by February 21st. It was also agreed that a follow-up working level meeting would be needed to discuss the details of PP&Ls supporting analyses after the RAI responses were received by the staff.

/S/

Chester Poslusny, Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-387/50-388

Enclosures:	1.	List of Attendees
	2.	Licensee's Handout

cc w/encls: See next page

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Pennsylvania Power & Light Company

Susquehanna Steam Electric Station, Units 1 & 2

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PP&L MEETING January 24, 1995

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ORGANIZATION

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Chet Poslusny
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Samir Serhan
Scott Boynton
Jay Lee

PP&L PP&L NRC/NRR Gilbert PP&L NRC NRC NRC NRC NRC NRC NRC PP&L PP&L PP&L GE (BWROG) Gilbert NRC NRC

ENCLOSURE 1

SUSQUEHANNA STEAM ELECTRIC STATION LICENSE AMENDMENT TO INCREASE MSIV LEAKAGE RATE AND DELETE THE MSIV LEAKAGE CONTROL SYSTEM

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► OVERVIEW

BENEFITS

- EVALUATIONS:
- LEAK RATE
- SEISMIC
- MODIFICATIONS
- ► SCHEDULE



- INCREASE ALLOWABLE MSIV LEAKAGE FROM 46 SCFH FOR ALL MAIN STEAM LINES TO 100 SCFH FOR ANY ONE MSIV WITH A TOTAL MAXIMUM LEAKAGE OF 300 SCFH FOR ALL FOUR MAIN STEAM LINES.
- ADD NEW REQUIREMENT TO REPAIR AND RETEST TO 11.5 SCFH FOR ANY VALVE NOT MEETING THE 100 SCFH CRITERIA.
- ► REMOVE MSIV-LCS AND UTILIZE MAIN STEAM DRAIN LINE AND CONDENSER
- ACCOMPLISH THE CHANGES IN ACCORDANCE WITH BWROG REPORT NEDC-31858P WHICH DETAILS HOW TO EVALUATE THE RADIOLOGICAL AND SEISMIC IMPLICATIONS.

BENEFITS

- REDUCE RADIATION EXPOSURE TO PLANT WORKERS.
- ► EXTEND EFFECTIVE SERVICE LIFE OF MSIV'S.
- ► MORE EFFECTIVE TREATMENT METHOD.
- ► EXPANDED OPERATING RANGE.
- ► NO LOGIC CONTROLS OR INTERLOCKS REQUIRED.
- ► MAINTAINS PHILOSOPHY OF MULTIPLE BARRIERS.
- ► REDUCE OUTAGE DURATION.

EVALUATIONS:LEAK RATE

RADIOLOGICAL EVALUATION:

- ► PERFORMED PER NEDC-31858P METHOD BY GE.
- ► SAME ASSUMPTIONS AND METHODS AS SSES FSAR.
- ► INCLUDES POWER UPRATE.

RADIOLOGICAL CONCLUSIONS:

- INSIGNIFICANT INCREASE TO PREVIOUS CALCULATED DOSES.
- SIGNIFICANT MARGIN TO DOSE LIMITS MAINTAINED.

EVALUATIONS:LEAK RATE

APPENDIX J CONFORMANCE

- ► CURRENT EXEMPTION BASED ON MSIV-LCS.
- WILL PROVIDE BASIS FOR TRANSFERRING THE EXEMPTION TO THE NEW TREATMENT METHOD AND LEAKAGE RATES BY 2/20/95 TO MEET NRC SCHEDULE NEEDS.

EVALUATIONS: • SEISMIC

- ► BWROG SEISMIC EXPERIENCE DATA BASE.
- ► SEISMIC BOUNDARY DETERMINATION BY PP&L.
- CONSTRUCTION CODE COMPARISON OF SSES TO SEISMIC EXPERIENCE DATA BASE.
- SEISMIC WALKDOWNS, OUTLIERS EVALUATED & MODIFIED AS NECESSARY.
- MARGIN ANALYSIS OF PIPING SYSTEM "WEAK, LINK" - EXPANSION ANCHOR
 - * SELECTED BASED ON JUDGMENT TO BE WORSE CASE.
 - * CONSERVATIVE, BIASED SAMPLE.
 - * ANALYSIS RESULTS CONFIRM THE WALKDOWN RESULTS.
- ► TURBINE BUILDING DESIGN.
- CONDENSER COMPARISON TO DATA BASE.
- ► CONDENSER ANCHORS ANALYZED.

<u>EVALUATIONS</u>:SEISMIC

SEISMIC CONCLUSIONS:

- MEET APPENDIX A
 - * EXPANSION ANCHOR SAMPLE.
 - * OUTLIER EVALUATIONS.
 - * EXPERIENCE DATA BASE.
 - * CONDENSER
 - * TURBINE BUILDING

► <u>MODIFICATIONS</u>

- ► ADDRESS OUTLIERS.
- PROVIDE DIESEL BACKED POWER TO ISOLATION VALVES AND DRAIN VALVE.
- ► REMOVE MSIV-LCS.

► <u>SCHEDULE</u>

- ► AMEND SUBMITTAL BY 2/20/95
 - * APPENDIX A
 - * APPENDIX J
- ► IMPLEMENT CHANGES DURING U27RIO IN 9/95.
- ► IMPLEMENT CHANGES DURING U19RIO IN 9/96.



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