



Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
600 Rocky Hill Road
Plymouth, MA 02360

Brian R. Sullivan
Site Vice President

May 25, 2017

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

SUBJECT: Supplement to Technical Specifications Proposed Change - Administrative Controls for Permanently Defueled Condition

Pilgrim Nuclear Power Station
Docket No. 50-293
Renewed License No. DPR-35

REFERENCES: Letter, Entergy Nuclear Operations, Inc., to NRC, "Technical Specifications Proposed Change - Administrative Controls for Permanently Defueled Condition," dated February 14, 2017 (Letter Number: 2.17.004)

LETTER NUMBER: 2.17.040

Dear Sir or Madam:

In accordance with Title 10 Code of Federal Regulations (CFR) 50.90, Entergy Nuclear Operations, Inc. proposed an amendment to Renewed Facility Operating License DPR-35 for Pilgrim Nuclear Power Station (PNPS) to address certain administrative controls that may be revised or removed to reflect the permanently defueled condition (Reference).

During U.S. Nuclear Regulatory Commission Staff review, four administrative discrepancies were noted on the retyped Technical Specifications pages. The purpose of this supplement is to address the following changes:

- Page iii Deleted "231" from list of Amendment numbers and drew a revision bar for Sections 5.6 and 5.7.
- Page 1-5 The proposed changes to page 1-4 moved the definition of PROTECTIVE FUNCTION from page 1-4 to page 1-5. Drew a revision bar next to page number 1-5.
- Page 5.0-4 Changed "S" in CERTIFIED FUEL HANDLERS to lower case.
- Page 5.0-12 Repositioned revision bar for change to align with the sentence that was modified.

ADD
NRR

PNPS has reviewed the administrative changes to the retyped pages and determined that the conclusion in the February 14, 2017 letter that the proposed changes do not involve a significant hazards consideration is not changed.

In accordance with 10 CFR 50.91, a copy of this supplement to the license amendment request, with attachments, is being provided to the Commonwealth of Massachusetts, Department of Public Health and Emergency Management Agency.

Attachment 1 to this letter provides a markup of Technical Specifications page 5.0-4 that was omitted in the original license amendment submission. Attachment 2 contains the retyped Technical Specifications pages reflecting the changes described above.

If you have any questions on this submittal, please contact Mr. Everett P. Perkins, Jr. at (508) 830-8323.

There are no new regulatory commitments made in this letter.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on May 25, 2017.

Sincerely,



Brian R. Sullivan
Site Vice President

BRS/pm

Attachments:

1. Markup of the Current Technical Specifications Page
2. Retyped Technical Specifications Pages

cc: Mr. Daniel H. Dorman
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U.S. Nuclear Regulatory Commission
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King of Prussia, PA 19406-2713

Mr. John Lamb, Project Manager
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Mr. John Priest, Director
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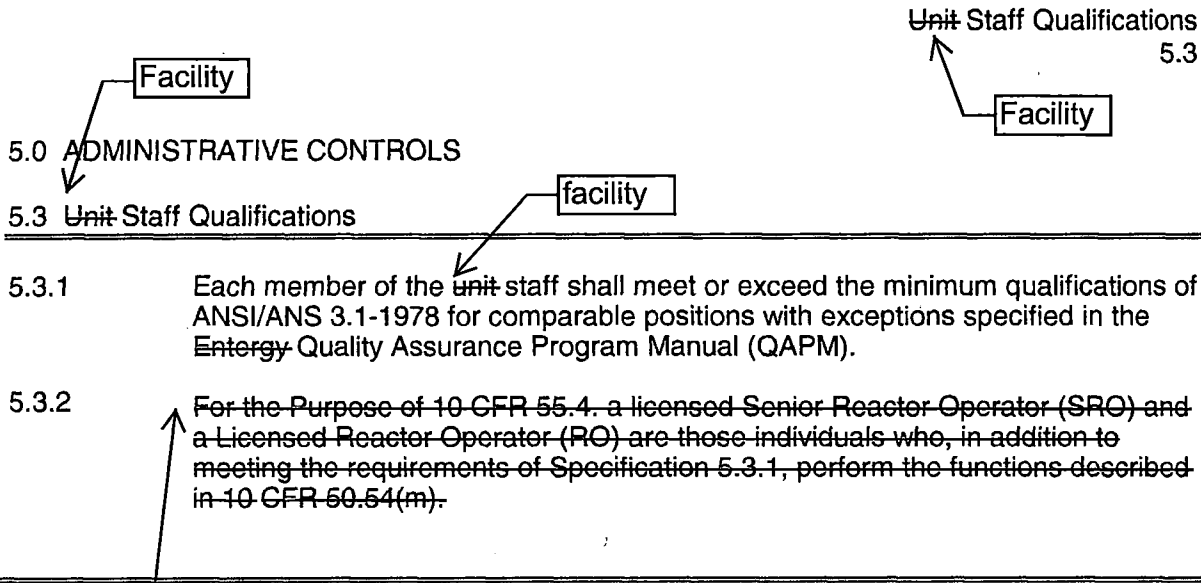
Mr. John Giarrusso, Jr.
Planning and Preparedness Section Chief
Mass. Emergency Management Agency
400 Worcester Road
Framingham, MA 01702

NRC Resident Inspector
Pilgrim Nuclear Power Station

Attachment 1

Letter Number 2.17.040

Pilgrim Nuclear Power Station
Markup of the Current Technical Specifications Page



An NRC approved training and retraining program for CERTIFIED FUEL HANDLERS shall be maintained.

Attachment 2

Letter Number 2.17.040

Pilgrim Nuclear Power Station
Retyped Technical Specifications Pages

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1.0 DEFINITIONS (continued)

PROTECTIVE FUNCTION	A system PROTECTIVE ACTION which results from the PROTECTIVE ACTION of the channels monitoring a particular plant condition.
REACTOR POWER OPERATION	REACTOR POWER OPERATION is any operation with the mode switch in the "Startup" or "Run" position with the reactor critical and above 1% design power.
REACTOR VESSEL PRESSURE	Unless otherwise indicated, REACTOR VESSEL PRESSURES listed in the Technical Specifications are those measured by the reactor vessel steam space detectors.
REFUELING INTERVAL	REFUELING INTERVAL applies only to In-service Code Testing Program surveillance tests. For the purpose of designating frequency of these code tests, a REFUELING INTERVAL shall mean at least once every 24 months.
REFUELING OUTAGE	REFUELING OUTAGE is the period of time between the shutdown of the unit prior to a refueling and the startup of the plant after that refueling. For the purpose of designating frequency of testing and surveillance, a REFUELING OUTAGE shall mean a regularly scheduled outage; however, where such outages occur within 11 months of completion of the previous REFUELING OUTAGE, the required surveillance testing need not be performed until the next regularly scheduled outage.
SAFETY LIMIT	The SAFETY LIMITS are limits below which the reasonable maintenance of the cladding and primary systems are assured. Exceeding such a limit is cause for unit shutdown and review by the Nuclear Regulatory Commission before resumption of unit operation. Operation beyond such a limit may not in itself result in serious consequences, but it indicates an operational deficiency subject to regulatory review.
SECONDARY CONTAINMENT INTEGRITY	SECONDARY CONTAINMENT INTEGRITY means that the reactor building is intact and the following conditions are met: <ol style="list-style-type: none">1. At least one door in each access opening is closed.2. The standby gas treatment system is operable.3. All automatic ventilation system isolation valves are operable or secured in the isolated position.
SIMULATED AUTOMATIC ACTUATION	SIMULATED AUTOMATIC ACTUATION means applying a simulated signal to the sensor to actuate the circuit in question.
SOURCE CHECK	A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.
STAGGERED TEST BASIS	A STAGGERED TEST BASIS shall consist of: (a) a test schedule for <u>n</u> systems, subsystems, trains, or other designated components obtained by dividing the specified test interval into <u>n</u> equal subintervals; (b) the testing of one system, subsystem, train or other designated components at the beginning of each subinterval.

5.0 ADMINISTRATIVE CONTROLS

5.3 Facility Staff Qualifications

- 5.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 for comparable positions with exceptions specified in the Quality Assurance Program Manual (QAPM).
- 5.3.2 An NRC approved training and retraining program for CERTIFIED FUEL HANDLERS shall be maintained.
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5.0 ADMINISTRATIVE CONTROLS

5.6 Reporting Requirements

The following reports shall be submitted in accordance with 10 CFR 50.4.

5.6.1 Not Used

5.6.2 Annual Radiological Environmental Operating Report

The Annual Radiological Environmental Operating Report covering the operation of the facility during the previous calendar year shall be submitted by May 15 of each year. The report shall include summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in the Offsite Dose Calculation Manual (ODCM), and in 10 CFR 50, Appendix I, Sections IV.B.2, IV.B.3, and IV.C.

The Annual Radiological Environmental Operating Report shall include a summary of the results of analyses of all radiological environmental samples and of all environmental radiation measurements taken during the period pursuant to the locations specified in the table and figures in the ODCM, as well as summarized and tabulated results of these analyses and measurements in the format of the table in the Radiological Assessment Branch Technical Position, Revision 1, November 1979. In the event that some individual results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted in a supplementary report as soon as possible.

(Continued)