

**LICENSEE EVENT REPORT (LER)**

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  
Susquehanna Steam Electric Station - Unit 1

DOCKET NUMBER (2)  
05000387

PAGE (3)  
1 OF 3

TITLE (4)  
Loss Of Continuous Vent Sampling - Unit 1 Reactor Building

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
4	26	98	98	-- 006	-- 00	5	18	98	FACILITY NAME	DOCKET NUMBER 05000
									FACILITY NAME	DOCKET NUMBER 05000

OPERATING MODE (9)	5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)								
POWER LEVEL (10)	000	20.2201(b)	20.2203(a)(2)(v)	X	50.73(a)(2)(i)	50.73(a)(2)(viii)				
		20.2203(a)(1)	20.2203(a)(3)(i)		50.73(a)(2)(ii)	50.73(a)(2)(x)				
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)		50.73(a)(2)(iii)	73.71				
		20.2203(a)(2)(ii)	20.2203(a)(4)		50.73(a)(2)(iv)	OTHER				
		20.2203(a)(2)(iii)	50.36(c)(1)		50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A				
		20.2203(a)(2)(iv)	50.36(c)(2)		50.73(a)(2)(vii)					

LICENSEE CONTACT FOR THIS LER (12)  
NAME: Cornelius T. Coddington - Senior Engineer, Licensing  
TELEPHONE NUMBER (Include Area Code): 717 / 542-3294

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE). X NO  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)  
On April 26, 1998, at 1000 hours, with Unit 1 in Condition 5 (Refueling) at 0 percent power, the Unit 1 Reactor Building Vent Monitoring System was declared inoperable due to tubing becoming disconnected from the sample flow and Technical Specification Limiting Conditions for Operation (LCOs) 3.3.7.5 and 3.3.7.11 were entered. Alternate sampling was established within 101 minutes from the receipt of the alarms. Current Technical Specifications does not allow for any time to establish alternate sampling upon the loss of normal sampling. The Unit 1 Reactor Building Vent Monitoring System was repaired (tubing inspected and reconnected) and returned to service on April 27, 1998. The cause of the event was determined to be that tubing for the sample flow lost resilience due to aging/end-of-life and separated from its connection. In addition, the design uses tubing connectors that do not clamp the tubing in place but rely on the elasticity of the tubing to grip the fitting. As the tubing loses its elasticity and grip on the fitting, the weight of the tubing and vibration can cause it to become disconnected. Analysis of the particulate filter and iodine cartridge which provided effluent sampling of the time immediately before and immediately after each of the interruptions in sampling, showed no abnormal activity, when averaged over the sample period. During the same periods of time, there were no changes in plant conditions or other indications of high levels of particulate or iodine activity which could cause any abnormal release during the interrupted flow time period. It is reasonable to conclude that no significant release occurred while the sampling was lost in this vent. As such, there were no consequences to the health and safety of the public. Tubing on all Vent Monitoring Systems is being replaced and a system reliability improvement plan is being implemented. A revision to the required action to allow a period of time to restore continuous sampling will be implemented in conjunction with the 1998 Improved Technical Specifications implementation plan.



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**TEXT CONTINUATION**

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Susquehanna Steam Electric Station - Unit 1	05000				2 OF 3
	387	98	-- 006	-- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**EVENT DESCRIPTION**

On April 26 1998, at 1000 hours with Unit 1 in Condition 5 (Refueling) at 0 percent power, the control room received several stack monitoring system trouble alarms. A Nuclear Plant Operator (NPO; utility; non-licensed) was dispatched to the local panel. The Unit 1 Reactor Building Vent Monitoring System (EIS Code: IL) was declared inoperable and Technical Specification Limiting Conditions for Operation (LCOs) 3.3.7.5 and 3.3.7.11 were entered. Alternate sampling was established within 101 minutes from the receipt of the alarms. The Unit 1 Reactor Building Vent Monitoring System was repaired (tubing reconnected) and returned to service on April 27, 1998.

**CAUSE OF EVENT**

The cause of the event was determined to be that tubing for the sample flow lost resilience due to aging/end-of-life and separated from its connection. In addition the design uses tubing connectors that do not clamp the tubing in place but rely on the elasticity of the tubing to grip the fitting. As the tubing loses its elasticity and grip on the fitting, the weight of the tubing and vibration can cause it to become disconnected.

**REPORTABILITY/ANALYSIS**

On April 26, 1998, from 0959 to 1140, continuous sampling from the Unit 1 Reactor Building vent was lost due to tubing becoming disconnected. The Technical Specification requirement to continuously sample this release point for particulate and iodine activity (Technical Specification 3.3.7.11 Action 112 and Technical Specification 4.11.2.1) could not be met during the 101 minutes it took to implement alternate continuous sampling. This event was determined to be reportable in accordance with 10CFR50.73(a)(2)(i)(B) in that there is no time period allowed to reestablish continuous sampling in the current Technical Specifications.

Analysis of the particulate filter and iodine cartridge which provided effluent sampling of the time immediately before and immediately after each of the interruptions in sampling, showed no abnormal activity, when averaged over the sample period. During the same periods of time, there were no changes in plant conditions or other indications of high levels of particulate or iodine activity which could cause any abnormal release during the interrupted flow time period. It is reasonable to conclude that no significant release occurred while the sampling was lost in this vent. As such, there were no consequences to the health and safety of the public.

In accordance with the guidelines provided in NUREG-1022, Revision 1, Section 5.1.1, the required submission date for this report was determined to be May 26, 1998.



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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

**CORRECTIVE ACTIONS**

The following corrective actions have been completed:

- The tubing was inspected and reconnected.
- The Reactor Building Vent Monitoring System was returned to service.

The following corrective actions are to be taken:

- The tubing in all the Vent Monitoring Systems will be replaced. (Tubing replacement began the week of May 11th and is scheduled to be completed by the end of June 1998.)
- Preventative Maintenance procedures will be implemented for the tubing.
- The plan to improve the reliability of the entire Ventilation Monitoring System will be implemented in accordance with the Maintenance Rule.

**ADDITIONAL INFORMATION**

Past Similar Events: Docket No. 50-387 LER 98-001-00  
LER 96-008-00  
LER 94-005-00  
LER 85-013-00  
LER 84-039-00

50-388 LER 98-006-00  
LER 98-003-00  
LER 98-002-00  
LER 97-005-00

Failed Component: None

