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 AUTH. NAME AUTHOR AFFILIATION
 BYRAM, R.G. Pennsylvania Power & Light Co.
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SUBJECT: Forwards changes to Bases of TS, revising existing TS Bases B3/4 9.7, to revise dose consequences from fuel handling accident & define methods of control of loads over spent fuel pool.

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PP&L, Inc.

Two North Ninth Street • Allentown, PA 18101-1179 • 610/774-5151

Robert G. Byram
Senior Vice President
Generation and Chief Nuclear Officer
610/774-7502
Fax: 610/774-5019

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U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
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**SUSQUEHANNA STEAM ELECTRIC STATION
CHANGE TO THE SSES TECHNICAL SPECIFICATION
BASES FOR TECHNICAL SPECIFICATION 3/4 9.7
PLA-4679 FILE R41-2**

Docket Nos. 50-387
and 50-388

The purpose of this letter is to transmit changes to the Bases of Susquehanna Unit 1 and 2 Technical Specifications. The changes revise the existing Technical Specification Bases B3/4 9.7, Crane Travel - Spent Fuel Storage Pool to revise the dose consequences from a fuel handling accident and to define the methods of the control of loads over the spent fuel pool.

BACKGROUND

Technical Specification 3/4 9.7 (Unit 1 and Unit 2) uses the term 'prohibited' but provides no guidance for the use of this term. This change provides guidance on the use of this term in the Unit 1 and Unit 2 Technical Specification Bases section 3/4 9.7. This change also revises the results of a fuel handling accident to be within the analysis presented in most recent fuel reload analysis.

DESCRIPTION OF CHANGE

Technical Specification 3/4.9.7 (Unit 1 and Unit 2) uses the term prohibited but provides no guidance on what type of restrictions can be used to restrict heavy loads over the spent fuel pools. This change provides guidance on the acceptable restrictions (as defined in the Licensing Bases) used to prohibit movement of heavy loads over the spent fuel pools in the Bases Section for Technical Specification 3/4.9.7. The following sentence has been added to the Bases Section:

'The restriction of movement is accomplished by interlocks, physical stops or administrative controls as stated in the Final Safety Analysis Report.'

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Pool
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This change also revises the consequences of a fuel handling accident as presented in the Bases Section of Technical Specification 3/4.9.7. This change is consistent with the latest approved reload analysis. The Bases Section has been revised to state the following:

'... (1) the dose consequences of the accident would be well within the requirements of 10CFR100 doses (less than 25% of 10CFR100 doses), and ...'

SAFETY ANALYSIS

PP&L has performed a 10 CFR 50.59 Safety Evaluation for this change. In summary:

The change does not have any effect on any accident scenario or malfunction of equipment important to safety nor does it create the possibility for an accident or malfunction. The change is administrative in nature in that it documents our existing program restrictions for movement of heavy loads in the vicinity of the spent fuel pool and also provides a revision of the dose consequence from a fuel handling accident consistent with the latest reload analysis. There is no effect upon operator performance from this change. There are no safety related elements associated with this change nor are there any impacts to the safety margin currently provided.


The revisions to the Technical Specification Bases have no adverse safety impact on SSES.

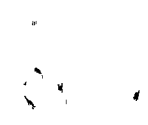
IMPLEMENTATION

PP&L has approved this change based upon review by the Plant Operations Review Committee. The attached revised pages are provided for your information and controlled distribution.

Questions regarding the above can be direct to Mr. C. T. Coddington at (717) 542-3294.

Very truly yours,


R.G. Byram
Attachments



copy: NRC Region I
Mr. K. M. Jenison, NRC Resident Inspector
Mr. C. Poslusny, NRC Sr. Project Manager
Mr. K. Kerns, Pa. DEP