



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 15, 1997

50-387/388

Mr. Robert G. Byram
Senior Vice President-Nuclear
Pennsylvania Power and Light
Company
2 North Ninth Street
Allentown, PA 18101

SUBJECT: NRC VISIT TO PENNSYLVANIA POWER AND LIGHT COMPANY OFFICE CONCERNING
OUTSTANDING ISSUE ON CLASS 1E/NON 1E ISOLATION AT SUSQUEHANNA STEAM
ELECTRIC STATION (SSES), UNITS 1 AND 2 (TAC NOS. M90541 AND M90542)

Dear Mr. Byram:

On April 29, 1997, I visited your office in Allentown, PA, with staff members from the Instrumentation and Controls Branch to audit information supporting your two submittals dated November 26, 1996 and February 3, 1997. These submittals provided analyses results to support the fact that isolators had not been installed in approximately 250 circuits in the SSES units. Each of the circuits included in the submittals reflect interface between Class 1E and non-Class 1E components. The purpose of the visit was to obtain a better understanding of the analyses assumptions, methods and conclusions and to determine the nature of the circuits that had been included.

During the discussions, the following facts were highlighted by your staff that could possibly lead to the resolution of the isolation issue for the SSES units: (1) approximately 150 of the analyzed circuits exclusively provide indication and do not interface with any protection system, and therefore, would not be subject to isolation specifications of IEEE-279-1971; (2) approximately 8 of the total circuits analyzed are those which have Class 1E and non-Class 1E interface and a fault or failure in the non-Class 1E device would cause the associated Class 1E component to fail safe, consistent with IEEE-279-1971; and (3) the remaining approximately 100 circuits have dual-element thermocouples that provide backup protection and are not credited in the plant safety analysis, therefore would not need isolation devices to meet IEEE-279-1971 requirements.

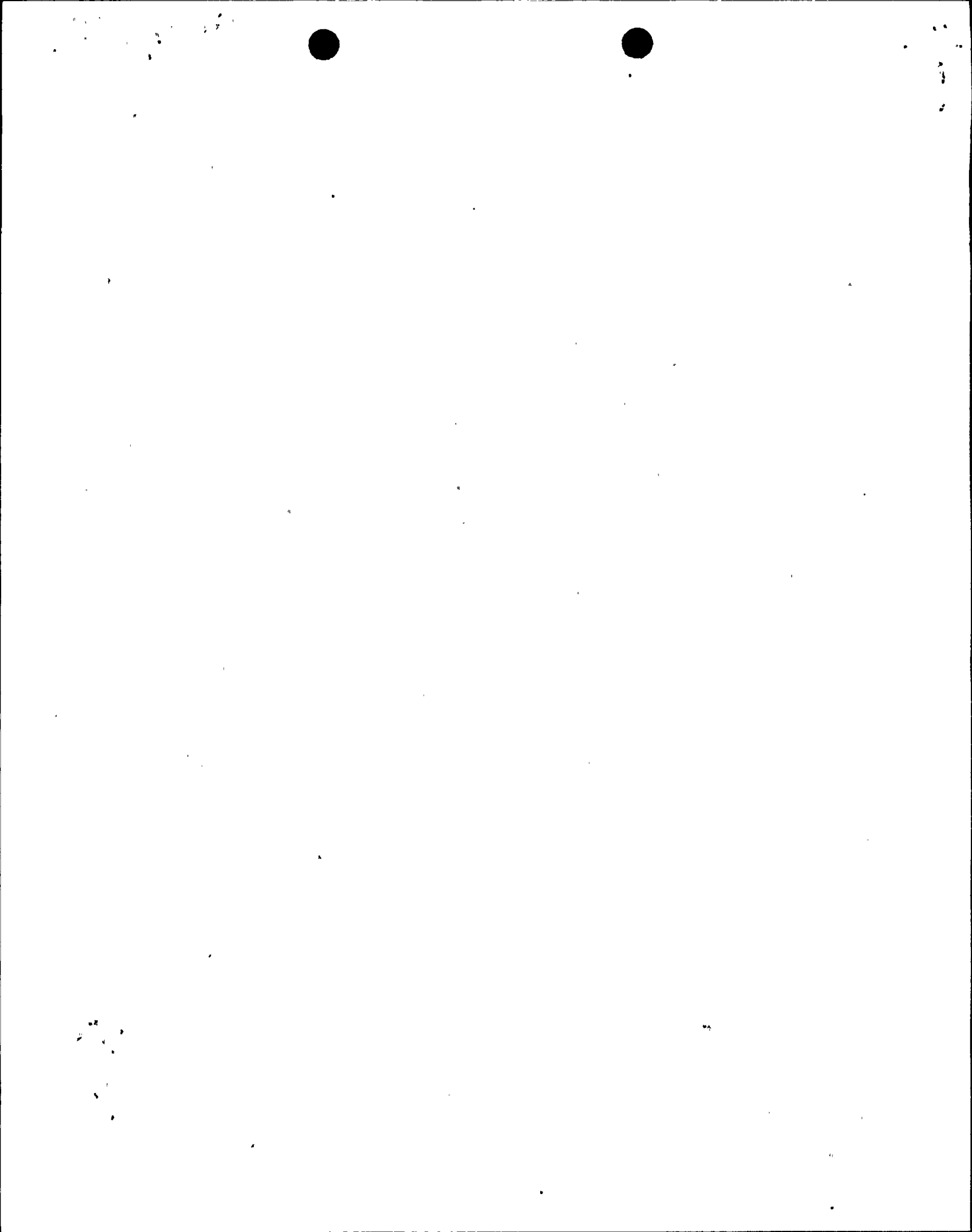
Based on the above clarification of the nature of the circuits that your staff has analyzed, the NRR technical staff suggested that PP&L provide to the Commission a document that would summarize the effort that was reflected in the two submittals and provide more details on items (1)-(3) above for staff review. The staff will then evaluate that information relative to plant compliance with IEEE-279-1971, as required by 10 CFR 50.55(a), and will develop a finding to be documented in a safety evaluation. Your staff indicated that it could provide such a submittal by the end of May 1997.

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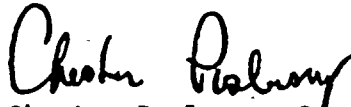
R. Byram

- 2 -

May 15, 1997

It would be appreciated if you could accelerate this submittal and ensure that it contains sufficient technical detail to facilitate a staff safety finding on this issue, which remains one of the oldest outstanding licensing issues with the Commission.

Sincerely,



Chester Poslusny, Senior Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-387/388

cc: See next page

R. Byram

- 2 -

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/s/

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Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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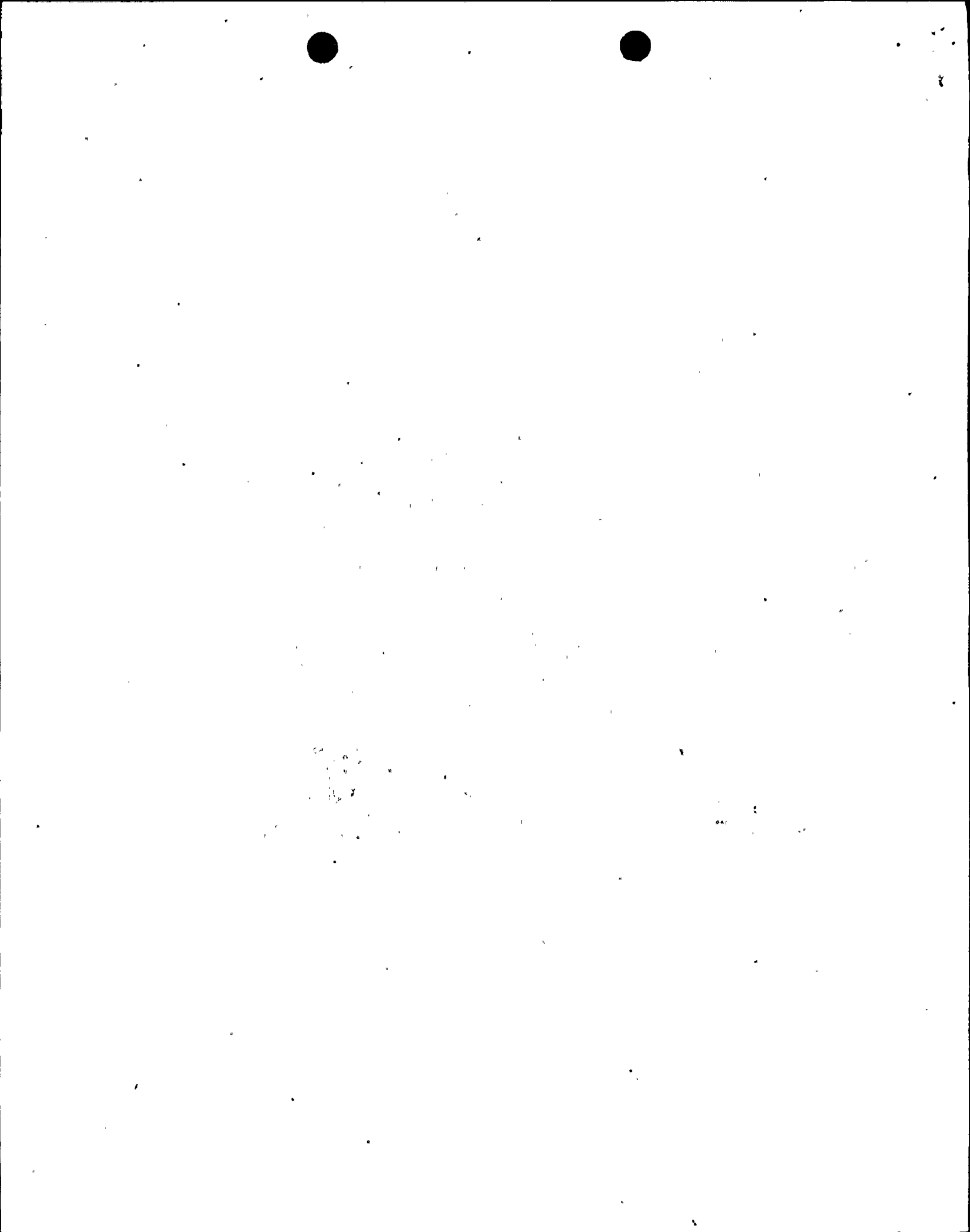
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Pennsylvania Power & Light Company

Susquehanna Steam Electric Station,
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