

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>Susquehanna Steam Electric Station - 1</b>	DOCKET NUMBER(2) <b>0 5 0 0 0 3 8 7 1</b>	PAGE (3) <b>OF 0 3</b>
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TITLE (4)  
**Fire Watch Rounds Not Completed Within Time Limits**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	0	2	9	7	9	7	9	Susquehanna SES - Unit 2		0 5 0 0 0 3 8 8
0	4	0	2	9	7	9	7	9			0 5 0 0 0 3 8 8

OPERATING MODE (9) <b>1</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR Y: (Check one or more of the following) (11)									
POWER LEVEL (10) <b>1 0 0</b>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)	OTHER (Specify in Abstract below and in Text, NRC Form 368A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(1)(2)(iv)(B)							
	<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)							

(LICENSEE CONTACT FOR THIS LER (12))

NAME <b>Comelius T. Coddington - Senior Project Engineer, Licensing</b>	TELEPHONE NUMBER
	AREA CODE: <b>7 1 7</b> NUMBER: <b>5 4 2 - 3 2 9 4</b>

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 2, 1997, with Unit 1 in Condition 1 (Power Operation) at 100% power and Unit 2 in Condition 5 (Refueling) at 0% power, it was discovered during line management review that, on two separate occasions, roving fire watch patrols did not survey fire areas as required by Technical Specification ACTION statement 3.7.7.a. These events are reportable per 10CFR50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications. The causes of the events were determined to be that the subject individuals' on-the-job fire watch training and qualification was ineffective and the methods used to document fire watch rounds did not provide timely feedback if an error should occur. The corrective actions include the survey of the areas missed, refresher training for fire watch personnel, revision to the on-the-job training practices, additional training for supervisors with respect to instructing and coaching, and evaluation of methods to provide prompt feedback to the fire watch patrol. Plant operation was not affected, and no fire occurred nor went undetected during the time period involved. As such, there were no safety consequences or compromises to public health and safety.

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PDR ADOCK 05000387  
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**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

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FACILITY NAME (1)  Unit 1  Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0   5   0   0   0   3   8   7, 9   7   —	LER NUMBER (6)						PAGE (3)		
		YEAR	SEQUENTIAL NUMBER			REVISION NUMBER				
		9   7	0	0	9	—	0	0	2	OF

TEXT (If more space is required, use additional NRC Form 366A's) (17)

**EVENT DESCRIPTION**

On April 2, 1997, at 1300 hours with Unit 1 in Condition 1 (Power Operation) at 100% power and with Unit 2 in Condition 5 (Refueling) at 0% power, a roving fire watch patrol during the 13:00 - 14:00 round failed to survey one fire area in the Control Structure within one (1) hour, as required by Technical Specification ACTION statement 3.7.7.a. At 2300 hours on April 2, 1997, another roving fire watch patrol during the 23:00 - 24:00 round failed to survey four (4) fire areas in the Unit 1 Reactor Building within the same ACTION statement time limit as described above. The missed fire watches were discovered during line management review of the fire watch documentation. The maximum time for which the one (1) hour time limit was exceeded for any area that was not surveilled was less than one (1) hour.

**CAUSE OF EVENT**

The causes of the events were determined to be: 1) the on-the-job fire watch training and qualification process was ineffective in that neither of the fire watch individuals (utility; non-licensed) involved had demonstrated competence to perform unaccompanied rounds for credit; and 2) the methods (log, Morse Watchman device, security door/computer, and plant labeling) used to document the fire watch rounds do not provide adequate feedback within the one-hour Technical Specification interval, so that an unaccompanied fire watch may detect and recover from an error of omission before a violation occurs.

**REPORTABILITY/ANALYSIS**

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), in that on two occasions on April 2, 1997, fire areas were not surveilled within the one-hour Technical Specification ACTION 3.7.7.a time requirement. This is a condition prohibited by Technical Specifications. On one of the occasions, the fire watch individual failed to survey one fire area in the Control Structure because an inoperable door introduced confusion in the normal patrol route. For the other occasion, the fire watch individual was disoriented as to the location and did not survey four (4) elevations in the Unit 1 Reactor Building. The maximum time the ACTION statement time limits were exceeded was less than one hour. Plant operation was not affected, and no fire occurred nor went undetected during the time period involved. As such, there were no safety consequences or compromises to public health and safety.

In accordance with the guidelines provided in NUREG-1022, Supplement 1, Items 14.1, the required submission date for this report was determined to be May 2, 1997.



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		9   7	—	0   0   9	—	0   0		3	OF	3

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**CORRECTIVE ACTIONS**

Corrective actions that were completed include the survey of all areas that were not surveilled, refresher training was initiated for all fire watch individuals, and supervisory oversight was increased.

The following corrective actions are to be implemented:

- Complete refresher training for all fire watch individuals.
- Implement a formal fire watch on-the-job training and requalification program which includes the establishment of specified proficiency requirements that regularly retest/requalify fire watch personnel.
- Train Effluents Management supervisors to be on-the-job training instructors and coaches.
- Evaluate the use and implement as appropriate more effective equipment that provides immediate feedback to fire watches, without interfering with their mobility, attentiveness or safety.

**ADDITIONAL INFORMATION**

Past Similar Events:

Docket No. 50-387 LER 96-001-00  
 LER 95-002-00  
 LER 94-002-00  
 LER 90-002-00

Failed Component: None