

# CATEGORY 1

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9704250256    DOC.DATE: 97/04/22    NOTARIZED: NO    DOCKET #  
FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv    05000388  
AUTH.NAME    AUTHOR AFFILIATION  
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RECIP.NAME    RECIPIENT AFFILIATION

SUBJECT: LER 97-005-00 on 970325, determined that reactor building vent continuous sample lost for 20 minutes attributed to unwarranted alarming of fuel pool criticality monitor. Restarted SPING sample pump. W/970422 ltr.

DISTRIBUTION CODE: IE22T    COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4  
TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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APR 22 1997

U.S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
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SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 50-388/97-005-00  
PLAS - 708 FILE R41-2

Docket No. 50-388  
License No. NPF-22

Attached is Licensee Event Report 50-388/97-005-00. This report is being made pursuant to 10CFR50.73(a)(2)(i)(B), in that Susquehanna Unit 2 was in a condition prohibited by Technical Specification Table 3.3.7.11, ACTION statement 112, when the Unit 2 Reactor Building SPING continuous sample was not maintained per the Technical Specification LCO.

  
G. J. Kuczynski  
General Manager - Susquehanna SES

Attachment

cc: Mr. H. J. Miller  
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Mr. Kenneth M. Jenison  
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PDR ADOCK 05000388  
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IE22/1

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U7.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2						DOCKET NUMBER(2) 0 5 0 0 0 3 8 8 1			PAGE (3) OF 0 3		
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TITLE (4)  
Reactor Building Vent Continuous Sample Lost For Twenty (20) Minutes

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 3	2 5	9 7	9 7	0 0 5	0 0	0 4	2 2	9 7	SSES - Unit 1		0 5 0 0 0 3 8 7
									0 5 0 0 0		

OPERATING MODE (9) 5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR Y : (Check one or more of the following) (11)										
POWER LEVEL (10) 0 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(b)							
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)							
	<input type="checkbox"/> 20.405(a)(1)(j)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
	<input type="checkbox"/> 20.405(a)(1)(k)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(v)(A)								
	<input type="checkbox"/> 20.405(a)(1)(l)	<input type="checkbox"/> 50.73(a)(2)(j)	<input type="checkbox"/> 50.73(1)(2)(v)(B)								
	<input type="checkbox"/> 20.405(a)(1)(m)	<input type="checkbox"/> 50.73(a)(2)(k)	<input type="checkbox"/> 50.73(a)(2)(x)								

(LICENSEE CONTACT FOR THIS LER (12))

NAME Stephen J. Ellis - Project Engineer, Licensing		TELEPHONE NUMBER	
		AREA CODE	
		7 1 7	5 4 2 - 3 5 3 7

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 25, 1997, at 1830 hours with Unit 2 in Condition 5 (Refuel) at 0% Power, the sample flow from the Reactor Building ventilation stack was lost for a period of 20 minutes. Chemistry technicians were in the process of transferring from an alternate sample system to the SPING system when a spurious criticality monitor alarmed, forcing the technicians to evacuate the area immediately. Some 20 minutes later, the technicians again accessed the area and completed the transfer placing the SPING in service. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), as a condition prohibited by the plant Technical Specification Table 3.3.7.11-1, ACTION statement 112, in that continuous sampling of the Unit 2 Reactor Building vent was not maintained. The cause of the event was determined to be the false alarming of the fuel pool criticality monitor. Upon determination that the alarm was false, the continuous sampling was promptly placed in service. Vent sampling immediately before and after the flow interruption were analyzed, and showed levels well below the LLD's referenced in Technical Specifications. Additionally, there were no changes in plant conditions that would be indicative of a release. Therefore, it was concluded that no significant release of radioactivity occurred while sampling was lost, and thus, there was no compromise to the health and welfare of the public. Corrective actions included restoration of the SPING sampling unit. An additional action will be to request from the NRC, clarification to Technical Specification 3.3.7.11-1, ACTION statement 112, for an interpretation of continuous sampling.



**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

FACILITY NAME (1)  Unit 2  Susquehanna Steam Electric Station	DOCKET NUMBER (2)  0   5   0   0   0   3   8   8	LER NUMBER (8)						PAGE (3)		
		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER				
		9   7	—	0   0   5	—	0   0	2	OF	3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

**EVENT DESCRIPTION**

On March 25, 1997, at 1830 hours with Unit 2 in Condition 5 (Refuel) at 0% power, the sample flow of the Reactor Building ventilation (EIS Code: IL) was lost for a 20 minute period. At the time of the loss, the technicians were in the process of transferring from an alternate sampling system, back to the SPING. While in the process of making the transfer, but before the Stationary Particulate Iodine Noble Gas (SPING) sample pump was started, the Spent Fuel Criticality Monitor alarmed, and the Chemistry technicians immediately evacuated the area. It was 20 minutes before the technicians were allowed to enter the area and complete the re-start of the Reactor Building SPING.

**CAUSE OF EVENT**

The cause of the event was attributed to the unwarranted alarming of the fuel pool criticality monitor, which required the Chemistry technicians to evacuate the area. It was found that the criticality monitor setpoint had drifted low. This, coupled with the on-going outage work, caused the monitor to alarm. After verification by Health Physics that the alarm was false, the Chemistry technicians proceeded to re-start the stack monitor sample pump.

**REPORTABILITY/ANALYSIS**

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), as a condition prohibited by the plant's Technical Specifications, in that Technical Specification Table 3.3.7.11-1, ACTION statement 112, was not met when the Unit 2 Reactor Building SPING was out-of-service with no alternate continuous sampling, for 20 minutes. The above referenced ACTION statement requires continuous sampling for particulate and iodine. Without continuous sampling, the particulate and iodine release from the Unit 2 Reactor Building during the time of the lost sampling are not definitely known, and would have to be estimated if an actual release occurred. Sampling immediately prior to and following the loss of sample flow showed only extremely low concentrations of some particulate activity when averaged over the sampling period. The concentrations observed are well below the LLD required in Technical Specifications. During the same period, there were no changes in plant conditions or other indications of high level of particulate or iodine activity which could cause any abnormal release during the 20 minute period. Also, as noted above, the criticality alarm was found to be false. Based on this analysis, there was no challenge to nuclear safety, and the health and welfare of the public was not compromised.



**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 2	DOCKET NUMBER (2) 0   5   0   0   0   3   8   8	LER NUMBER (6)						PAGE (3)		
		YEAR 9   7	SEQUENTIAL NUMBER —	SEQUENTIAL NUMBER 0   0   5	REVISION NUMBER —	REVISION NUMBER 0   0	REVISION NUMBER 0   0	3	OF	3
Susquehanna Steam Electric Station	0   5   0   0   0   3   8   8	9   7	—	0   0   5	—	0   0	3	OF	3	

TEXT (if more space is required, use additional NRC Form 366A's) (17)

**CORRECTIVE ACTIONS**

The following corrective actions have been identified and completed:

- The SPING sample pump was restarted, and the vent monitor returned to service.
- The Fuel Pool Criticality Monitor was recalibrated.

A request for interpretation will be written and submitted to the NRC to obtain an interpretation of alternate continuous sampling and to clarify whether any allowance can be applied to "continuous sampling" to allow response to unplanned events.

**ADDITIONAL INFORMATION**

Failed Component: None

Past Similar Events: LER 50-387/84-039  
LER 50-387/85-013  
LER 50-387/94-005-00  
LER 50-387/96-008-00