

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

14. ANF-1125(P)(A) and ANF-1125(P)(A), Supplement 1, "ANFB Critical Power Correlation," April 1990.
15. NEDC-32071P, "SAFER/GESTR-LOCA Loss of Coolant Accident Analysis," GE Nuclear Energy, May 1992.
16. NE-092-001A, Revision 1, "Licensing Topical Report for Power Uprate With Increased Core Flow," Pennsylvania Power & Light Company, December 1992.
17. NRC SER on PP&L Power Uprate LTR (November 30, 1993).
18. PL-NF-90-001, Supplement 1-A, "Application of Reactor Analysis Methods for BWR Design and Analysis: Loss of Feedwater Heating Changes and Use of RETRAN MOD 5.1," September 1994.
19. PL-NF-94-005-P-A, "Technical Basis for SPC 9cd-2 Extended Fuel Exposure at Susquehanna SES," January 1995.
20. NEDE-24011-P-A-10, "General Electric Standard Application for Reactor Fuel," February 1991. **(-A)**
21. PL-NF-90-001, Supplement 2, "Application of Reactor Analysis Methods to BWR Design and Analysis: CASCRO-98 Code and ANFB Critical Power Correlation."

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6.9.3.3 The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, transient analysis limits and accident analysis limits) of the safety analysis are met.

6.10. RECORD RETENTION

In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.1 The following records shall be retained for at least 5 years:

- a. Records and logs of unit operation covering time interval at each power level.

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SUSQUEHANNA - UNIT 2

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TEL: 301-504-5369

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22. ANF-89-98(P)(A) Revision 1 and Revision 1 Supplement 1, "Generic Mechanical Design Criteria for BWR Fuel Designs," Advanced Nuclear Fuels Corporation, May 1995.
23. XN-NF-81-58 (P)(A) Supplements 1 and 2 Revision 2, "RODEX 2 Fuel Rod Thermal-Mechanical Response Evaluation Model," May 1986.
24. XN-NF-85-74(P)(A), "RODEX 2A (BWR) Fuel Rod Thermal-Mechanical Response Evaluation Model," August 1986.
25. XN-NF-82-06(P)(A) and Supplements 2, 4, and 5 Revision 1, "Qualification of Exxon Nuclear Fuel for Extended Burnup," October 1986.
26. XN-NF-85-92(P)(A), "Exxon Nuclear Uranium Dioxide/Gadolinia Irradiation Examination and Thermal Conductivity," November 1986.
27. ANF-90-082(P)(A) Revision 1 and Revision 1 Supplement 1, "Application of ANF Design Methodology for Fuel Assembly Reconstitution," May 1995.
28. ANF-91-048(P)(A), "Advanced Nuclear Fuels Corporation Methodology for Boiling Water Reactors EXEM BWR Evaluation Model," January 1993.
29. ANF-CC-33(P)(A) Supplement 2, "HUXY : A Generalized Multirod Heatup Code with 10CFR50 Appendix K Heatup Option," January 1991.
30. ANF-CC-33(P)(A) Supplement 1 Revision 1, "HUXY : A Generalized Multirod Heatup Code with 10CFR50 Appendix K Heatup Option Users Manual," August 1986: NOVEMBER 1975
31. XN-NF-80-19(P)(A), Volumes 2, 2A, 2B, and 2C "Exxon Nuclear Methodology for Boiling Water Reactors: EXEM BWR ECCS Evaluation Model," September 1982.
32. XN-NF-80-19(P)(A), Volumes 3 Revision 2 "Exxon Nuclear Methodology for Boiling Water Reactors Thermex: Thermal Limits Methodology Summary Description," January 1987.
33. XN-NF-79-71(P)(A) Revision 2, Supplements 1, 2, and 3, "Exxon Nuclear Plant Transient Methodology for Boiling Water Reactors," March 1986.
34. ANF-1358(P)(A), Revision 1, "The Loss of Feedwater Heating Transient in Boiling Water Reactors," September 1992.
35. ANF-913(P)(A) Volume 1 Revision 1 and Volume 1 Supplements 2, 3, and 4, "COTRANSA2 : A Computer Program for Boiling Water Reactor Transient Analyses," August 1990.



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