

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

·ACCESSION NBR:9612260061 DOC.DATE: 96/12/17 NOTARIZED: NO DOCKET #
 FACIL:50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylva 05000387
 AUTH.NAME AUTHOR AFFILIATION
 CODDINGTON,C.T. Pennsylvania Power & Light Co.
 KUCZYNSKI,G.J. Pennsylvania Power & Light Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 96-016-00:on 961118,'B'loops of both RHR & core spray
 sys inoperable occurred.Cause of event operator did not pay
 attention to detail & failed to operate valve in accordance
 with procedures.W/961217 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: 05000387G

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	RES/DET/EIB	1 1	RGN1 FILE 01	1 1	D
EXTERNAL:	L ST LOBBY WARD	1 1	LITCO BRYCE,J H	1 1	O
	NOAC MURPHY,G.A	1 1	NOAC POORE,W.	1 1	C
	NRC PDR	1 1	NUDOCS FULL TXT	1 1	U
NOTES:		1 1			M

NOTE TO ALL "RIDS" RECIPIENTS:
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Pennsylvania Power & Light Company

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December 17, 1996

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 50-387/96-016-00
PLAS - 691 FILE R41-2

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 50-387/96-016-00. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B) in that both 'B' loop of the Core Spray System and 'B' loop of the Residual Heat Removal System were inoperable at the same time. This is a condition prohibited by the Technical Specifications.

G. J. Kuczynski
Plant Manager - Susquehanna SES

Attachment

cc: Mr. H. J. Miller
Regional Administrator
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Kenneth M. Jenison
Sr. Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 35
Berwick, PA 18603-0035

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PDR ADOCK 05000387
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 1						DOCKET NUMBER(2) 0 5 0 0 0 3 8 7 1			PAGE (3) OF 0 3		
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TITLE (4)
'B' Loops of Both RHR and Core Spray Systems Inoperable

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	1	1 8 9 6	9 6	0 1 6	0 0	1	2 1 7 9 6				0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR Y : (Check one or more of the following) (11)										
POWER LEVEL (10) 1 0 0	20.402(b)	20.405(c)	50.73(a)(2)(v)	73.71(b)							
	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)							
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(v)								
	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(iv)(A)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(1)(2)(iv)(B)								
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(v)								

(LICENSEE CONTACT FOR THIS LER (12))

NAME Comellus T. Coddington - Sr. Project Engineer	AREA CODE 7 1 7	TELEPHONE NUMBER 5 4 2 - 3 2 8 9
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On November 18, 1996, at 0140, with Unit 1 in Condition 1 (Power Operation) at 100% power, both the 'B' loop of Residual Heat Removal (RHR) System and the 'B' loop of the Core Spray (CS) System were declared inoperable due to low keepfill pressure during the removal of the Condensate Transfer System for planned maintenance. With both the 'B' loops of the RHR and Core Spray Systems inoperable, Technical Specification 3.0.3 was entered. This event is reportable under 10CFR50.73(a)(2)(i)(B) since it is a condition prohibited by Technical Specifications. During the evolution of lining up the Refuel Water Transfer pumps to the Condensate Transfer System to provide keepfill during planned maintenance on the Condensate Transfer System, the keepfill pressure in both 'B' loops of RHR and Core Spray dropped below the pressure required for the operability of these systems. A Nuclear Plant Operator (NPO) was dispatched to check the lineup for Refuel Water Transfer to Condensate Transfer. The NPO discovered that a valve was in the incorrect position. The valve was closed when it should have been open. The NPO opened the valve and keepfill pressure was restored. Technical Specification 3.0.3 was exited at 0148. The cause of the event was determined to be that an operator did not pay attention to detail and failed to operate the valve in accordance with procedures. The corrective actions include having the operator review the operations procedure on operations policies and work practices, in particular, the section on valve position confirmation and operation and completing upgrade training on Pennsylvania Power & Light Company's program on self checking. There were no safety consequences or compromises to public health and safety as a result of declaring the 'B' loops of RHR and Core Spray inoperable since the 'A' loops of these systems were operable and would have mitigated a design basis event.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	LER NUMBER (6)						PAGE (3)		
		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER				
		9 6	—	0 1 6	—	0 0	2	OF	3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

On November 18, 1996, at 0140, with Unit 1 in Condition 1 (Power Operation) at 100% power, both the 'B' loop of Residual Heat Removal (RHR; EISS Code: BO) System and the 'B' loop of the Core Spray (CS; EISS Code: BM) System were declared inoperable due to low keepfill pressure during the removal of the Condensate Transfer System (EISS: Code KA) for planned maintenance. With both the 'B' loops of the RHR and Core Spray Systems inoperable, Technical Specification 3.0.3 was entered. This event is reportable under 10CFR50.73(a)(2)(i)(B) since it is a condition prohibited by Technical Specifications. During the evolution of lining up the Refuel Water Transfer pumps to the Condensate Transfer System to provide keepfill during planned maintenance on the Condensate Transfer System, the keepfill pressure in both 'B' loops of RHR and Core Spray dropped below the pressure required for the operability of these systems. A Nuclear Plant Operator (NPO; utility, non-licensed) was dispatched to check the lineup for Refuel Water Transfer to Condensate Transfer. The NPO discovered that a valve was in the incorrect position. The valve was closed when it should have been open. The NPO opened the valve and keepfill pressure was restored. Technical Specification 3.0.3 was exited at 0148.

CAUSE OF EVENT

The cause of the event was determined to be that the operator did not pay attention to detail and failed to operate the valve in accordance with procedures.

REPORTABILITY/ANALYSIS

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), in that Susquehanna SES Unit 1 was in a condition prohibited by the Technical Specifications when both 'B' loops of the RHR and Core Spray systems were declared inoperable due to low keepfill pressure. If an auto start signal had been received while keepfill pressure was low, the RHR and Core Spray systems would not have catastrophically failed but would have functioned to provide water to the reactor vessel to mitigate the event. Physical damage to pipe hangers could have possibly occurred due to the sudden increase in pressure. In addition, the 'A' loops of the RHR and Core Spray systems were operable and would have mitigated a design basis event. Therefore, there were no safety consequences or compromises to the public health or safety.

In accordance with the guidelines provided in NUREG-1022, Supplement 1, Item 14.1, the required submission date for this report was determined to be December 18, 1996.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)					
		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER							
Unit 1 Susquehanna Steam Electric Station	0 5 0 0 0 3 8 7	9	6	—	0	1	6	—	0	0	3	OF	3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTIONS

The following corrective actions have been identified:

- Have the operator review the operations procedure on operations policies and work practices in particular the section on valve position confirmation and operation.
- Complete upgrade training on Pennsylvania Power & Light Company's program on self checking.

ADDITIONAL INFORMATION

Past Similar Events: Docket No. 50-387 LER 96-004-00

Failed Component: None