

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9611250116 DOC. DATE: 96/11/15 NOTARIZED: NO DOCKET #
 FACIL: 50-387 Susquehanna Steam Electric Station, Unit 1, Pennsylvania 05000387
 AUTH. NAME AUTHOR AFFILIATION
 WEHRY, R.R. Pennsylvania Power & Light Co.
 KUCZYNSKI, G.L. Florida Power & Light Co.
 RECIPIENT NAME RECIPIENT AFFILIATION

SUBJECT: LER 96-013-00: on 961019, mode change w/RHR 'B' loop of
 LPCI was declared inoperable. Caused by human error. Event was
 reviewed w/all licensed operators during Training cycle
 96-6 & Unit Supervisor was counseled. W/961115 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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Pennsylvania Power & Light Company

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November 15, 1996

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
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SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 50-387/96-013-00
PLAS - 688 FILE R41-2

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 50-387/96-013-00. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B) in that Unit 1 entered Condition 2 (Startup) with the 'B' Loop of Residual Heat Removal system inoperable for the Low Pressure Coolant Injection mode. This constituted a condition prohibited by the plant's Technical Specifications Limiting Condition for Operation 3.0.4.


G. J. Kuczynski
Plant Manager - Susquehanna SES

Attachment

cc: Mr. H. J. Miller
Regional Administrator
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Mr. Kenneth M. Jenison
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PDR ADOCK 05000387
S PDR

JE22/1

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE: INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Susquehanna Steam Electric Station - Unit 1

DOCKET NUMBER(2)

0 5 0 0 0 3 8 7 1 OF 0 3

PAGE (3)

TITLE (4)

Mode Change With RHR 'B' Loop of LPCI Inoperable - Condition Prohibited By The Technical Specifications

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)							
1	0	1	9	9	6	9	6	0	1	3	0	5	0	0	0			

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 1. (Check one or more of the following) (11)														
2		20.402(b)			20.405(c)			50.73(a)(2)(v)			73.71(b)					
POWER LEVEL (10)		20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)					
0 0 0		20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
		20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(vii)(A)								
		20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(1)(2)(viii)(B)								
		20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)								

NAME															TELEPHONE NUMBER				
Richard R. Wehry - Nuclear Licensing Engineer															AREA CODE				
															7 1 7 5 4 2		- 3 6 6 4		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR	
<input type="checkbox"/> YES (if yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO																	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0036 hours on October 19, 1996, with Unit 1 in Condition 4 (Cold Shutdown) in preparation for starting up following its ninth refueling outage, the reactor mode switch was taken to Startup (Condition 2). At approximately 0045 hours, it was identified that the mode change to Condition 2 had been made with the 'B' Loop of Residual Heat Removal (RHR) system inoperable for the Low Pressure Coolant Injection (LPCI) mode. Two LPCI subsystems are required to be OPERABLE in Condition 2 per Technical Specification LCO 3.5.1. Since only one subsystem (RHR 'A' Loop) was OPERABLE, this constituted a condition prohibited by the plant's Technical Specifications LCO 3.0.4 which states that entry into an Operational Condition shall not be made when the conditions for the LCOs are not met. The root cause of this event was attributed to human error by the Unit Supervisor. The 'B' Loop of RHR was declared operable for the LPCI mode at 0120 hours on 10/19/96. Corrective actions include: counseling of the involved Unit Supervisor; review of this event with all operations personnel; procedural enhancements; and evaluation of a modification to better facilitate aligning the RHR system into the LPCI mode of operation.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 1 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 7	LER NUMBER (6)						PAGE (3)		
		YEAR 9 6	SEQUENTIAL NUMBER — 0 1 3	REVISION NUMBER — 0 0	2	OF	3			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

At 0036 hours on October 19, 1996, with Unit 1 in Condition 4 (Cold Shutdown) in preparation for starting up following its ninth refueling outage, the reactor mode switch was taken to Startup (Condition 2). At approximately 0045 hours on 10/19/96, upon opening of the Residual Heat Removal (RHR) 'B' and 'D' pump motor breaker 125 VDC control power knife switches, it was recognized that the mode change from Cold Shutdown (Condition 4) to Startup (Condition 2) had been made with the 'B' Loop of RHR system inoperable for the Low Pressure Coolant Injection (LPCI; EIS Code: BO) mode. Technical Specification LCO 3.5.1 requires two subsystems of RHR LPCI mode to be OPERABLE in Condition 2. Only one subsystem ('A' RHR Loop) was OPERABLE at the time.

CAUSE OF EVENT

The root cause of this event was attributed to human error. Alignment of the RHR 'B' Loop into the LPCI mode of operation required opening of the RHR 'B' and 'D' pump motor breaker 125 VDC control power knife switches. The Unit Supervisor (utility; licensed) determined that an LCO was not required for the RHR pumps because the unit was in Condition 4 (Cold Shutdown) where RHR LPCI mode is not applicable. This thought process contributed to his losing focus on the requirement of Technical Specification LCO 3.0.4 as it pertained to RHR LPCI operability and the mode change from Condition 4 to Condition 2. Technical Specification LCO 3.0.4 states that entry into an Operational Condition shall not be made when the conditions for the LCOs are not met.

REPORTABILITY / ANALYSIS

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B) in that Unit 1 entered Condition 2 (Startup) with the 'B' Loop of the RHR system inoperable for the LPCI mode of operation. Technical Specification LCO 3.5.1 requires two subsystems of RHR LPCI mode to be OPERABLE in Condition 2. Only one subsystem ('A' RHR Loop) was OPERABLE at the time. This constituted a condition prohibited by the plant's Technical Specifications LCO 3.0.4, which states that entry into an Operational Condition shall not be made when the conditions for the LCOs are not met. There were no safety consequences or compromises to public health and safety as a result of this event. There were no reactor vessel level perturbations or LPCI initiation signals during the period of time (44 minutes) in which the 'B' Loop of RHR was not aligned in the LPCI mode. Additionally, during the subject time period, operators were in the process of restoring the 'B' Loop of RHR to the LPCI mode of operation. The 'A' Loop of RHR was OPERABLE in the LPCI mode during the subject time period.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

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		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER				
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TEXT (if more space is required, use additional NRC Form 366A's) (17)

In accordance with the guidelines provided in NUREG 1022, Supplement 1, Item 14.1, the required submission date for this report was determined to be November 18, 1996.

CORRECTIVE ACTIONS

The condition was recognized at approximately 0045 hours on 10/19/96 as the 'B' Loop RHR pump motor breakers 125 VDC control power knife switches were being opened during restoration of the Loop to the LPCI mode of operation. The restoration was completed at 0120 hours on 10/19/96. The subject Unit Supervisor was counseled, the event was reviewed with all operations personnel and procedural enhancements were made to the operating procedure which controls the plant mode change from Condition 4 (Cold Shutdown) to Condition 2 (Startup). Additional actions to prevent recurrence include: review of this event with all licensed operators during Training Cycle 96-6; and an evaluation to determine if cost beneficial modifications are appropriate to better facilitate aligning the RHR system into the LPCI mode of operation.

ADDITIONAL INFORMATION

Failed Component Identification: Not applicable

Previous Similar Events: LERs 50-387 / 93-014-00 and 90-030-00 described events in which entries were made into an Operational Condition prior to having completed all necessary surveillance requirements.