

CATEGORY 1

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9611050304 DOC. DATE: 96/10/30 NOTARIZED: NO DOCKET #
 FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylvania 05000388
 AUTH. NAME AUTHOR AFFILIATION
 WEHRY, R.R. Pennsylvania Power & Light Co.
 KUCZYNSKI, G.J. Pennsylvania Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 96-007-00: on 960930, declared secondary containment damper inoperable. Caused by ineffective communication. Reviewed event w/personnel & re-emphasized expectations for performance. W/961030 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 5
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101-1179 • 610/774-5151

October 30, 1996

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 50-388/96-007-00
PLAS - 686 FILE R41-2

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 50-388/96-007-00. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B) in that a Technical Specification Limiting Condition for Operation ACTION was not complied with on Susquehanna Unit 2 for an inoperable secondary containment isolation damper that affected operation of both Susquehanna Unit 1 and Unit 2.


G. J. Kuczyński
Plant Manager - Susquehanna SES

Attachment

cc: Mr. H. J. Miller
Regional Administrator
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Kenneth M. Jenison
Sr. Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 35
Berwick, PA 18603-0035

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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2										DOCKET NUMBER (2) 0 5 0 0 0 3 8 8 1				PAGE (3) OF 0 4	
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TITLE (4)
Condition Prohibited By Plant's Technical Specifications Due To Inoperable Secondary Containment Isolation Damper

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)									
0	9	3	0	9	6	9	6	0	0	7	0	0	1	0	3	8	8	1	0	4
										SSES - Unit 1		0 5 0 0 0 3 8 7								

OPERATING MODE (9) 1

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR Y: (Check one or more of the following) (11)

20.402(b)	<input type="checkbox"/>	20.405(c)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	73.71(b)	<input type="checkbox"/>
20.405(a)(1)(i)	<input type="checkbox"/>	50.36(c)(1)	<input type="checkbox"/>	60.73(a)(2)(v)	<input type="checkbox"/>	73.71(c)	<input type="checkbox"/>
20.405(a)(1)(ii)	<input type="checkbox"/>	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(w)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 365A)	
20.405(a)(1)(iii)	<input checked="" type="checkbox"/>	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(x)(A)	<input type="checkbox"/>		
20.405(a)(1)(iv)	<input type="checkbox"/>	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(1)(2)(x)(B)	<input type="checkbox"/>		
20.405(a)(1)(v)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(x)	<input type="checkbox"/>		

(LICENSEE CONTACT FOR THIS LER (12))

NAME Richard R. Wehry - Nuclear Licensing Engineer		TELEPHONE NUMBER	
		AREA CODE	
		7 1 7	5 4 2 - 3 6 6 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1500 hours an October 1, 1996, with Unit 2 in Condition 1 (Power Operation) at 100% power, Operations personnel discovered, during work document reviews, that a condition prohibited by the plant's Technical Specifications had occurred during a work evolution on a secondary containment isolation damper. Namely, work had been performed on a Unit 1 secondary containment zone III isolation damper that affected operation of both Susquehanna Unit 1 and Unit 2. Unit 1 was in its ninth refueling outage with no core alterations in progress at the time. Therefore, secondary containment zone III was not required for Unit 1. Following completion of the work, the personnel protective blocking had been cleared and power restored to the damper solenoid at 0615 hours on 9/30/96. Since operability testing had not yet been performed, the damper remained inoperable for Unit 2, but this was not recognized until the work document review on 10/1/96 at 1500 hours, which was in excess of the 8 hour LCO ACTION time requirement of Technical Specification 3.6.5.2. The root causes of this event were attributed to failure of the Unit Coordinator to effectively communicate that work being performed on the subject damper impacted both Unit 1 and Unit 2; failure of the Operations Work Control Center Unit Supervisor (Senior Reactor Operator) to recognize the impact of the inoperable damper on Unit 2 operation; and the work release program not requiring the authorization of the opposite unit's Unit Supervisor to release work on equipment affecting both units. Corrective actions include: reviewing this event and its significance with all involved personnel; reviewing this event with all operations personnel to include emphasizing that work involving common equipment be discussed with both Unit Supervisors; and establishing necessary programmatic requirements for obtaining authorization by the opposite unit's Unit Supervisor when releasing common equipment for work.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Unit 2 Susquehanna Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 8	LER NUMBER (6)						PAGE (3)		
		YEAR 9 6	SEQUENTIAL NUMBER — 0 0 7	REVISION NUMBER — 0 0	2	OF	4			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

At 1500 hours on October 1, 1996, with Unit 2 in Condition 1 (Power Operation) at 100% power and Unit 1 in its ninth refueling outage, Operations personnel discovered during a work document review, that a condition prohibited by the plant's Technical Specifications had occurred during the work evolution involving a Unit 1 secondary containment isolation damper (EISS Code: VA). Namely, work had been performed on a Unit 1 secondary containment zone III isolation damper that affected operation of both Susquehanna Unit 1 and Unit 2, since this damper serves to isolate the common refueling floor from outside air for events which require the Standby Gas Treatment System (EISS Code: BH) to function. Unit 1 was in its ninth refueling outage with no core alterations in progress. Therefore, secondary containment zone III was not required for Unit 1. Secondary containment zone III was required, however, for Unit 2. The damper was inoperable and deactivated from 1105 on 9/29/96 to 0615 on 9/30/96, but was inoperable, and not deactivated, from 0615 hours on 9/30/96 until 1752 hours on 10/1/96. The second time frame represented a condition prohibited by Technical Specification Limiting Condition for Operation (LCO) 3.6.5.2 for Unit 2, which requires that the damper be restored to OPERABLE status or that the penetration be isolated by at least one deactivated damper secured in the isolation position within eight (8) hours.

CAUSE OF EVENT

The root causes of this event were attributed to:

- The failure of the Unit Coordinator to effectively communicate that work being performed on the subject Unit 1 damper impacted both Unit 1 and Unit 2;
- The failure of the Operations Work Control Center Unit Supervisor (Senior Reactor Operator) to recognize the impact to Unit 2 of the subject Unit 1 damper work evolution;
- The work release program did not require authorization of the opposite unit's Operations Unit Supervisor for release of work on equipment affecting both Unit 1 and Unit 2.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

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		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER				
		9 6	—	0 0 7	—	0 0		3	OF	4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

REPORTABILITY / ANALYSIS

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B) in that Technical Specification 3.6.5.2 LCO ACTION was not complied with on Susquehanna Unit 2 for an inoperable secondary containment isolation damper that affected operation of both Susquehanna Unit 1 and Unit 2. There were no safety consequences or compromises to public health and safety as a result of this event. The subject damper was not called upon to perform its specified safety function during the time it was inoperable. Work performed on the damper did not inhibit the station's ability to maintain the required differential pressure in secondary containment zone III. Secondary containment zone III isolation dampers isolate the refueling floor from outside air for events which require the Standby Gas Treatment System to function. The other division's damper in the same penetration remained operable throughout the time that the subject damper was inoperable and would have isolated secondary containment. Although not deactivated as required by the Technical Specification ACTION, the subject damper remained closed with a status control tag applied to its control switch during the time period from 0615 hours on 9/30/96 until 1752 hours on 10/1/96. As such, the damper would have remained in its specified safety function position for any design basis event.

In accordance with the guidelines provided in NUREG- 1022, Supplement 1, Items 14.1 and 14.5, the required submission date for this report was determined to be October 31, 1996.

CORRECTIVE ACTIONS

The following actions were completed:

- The event was reviewed with the Unit Coordinator who prepared the damper work release information and with all Unit Coordination Unit Coordinators.
- The expectations for flawless performance were re-emphasized to Unit Coordination / Plant Scheduling organization.
- The Operations Work Control Center Unit Supervisor was counseled on the proper method for releasing equipment for work that impacts both units.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

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		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER				
		9 6	—	0 0 7	—	0 0		4	OF	4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The following actions to prevent recurrence are being taken:

- The event will be reviewed with all Operations personnel, including emphasizing review of all applicable Technical Specifications and Technical Specification interpretations.
- The necessary programmatic requirements will be established for obtaining authorization by the opposite unit Operations Unit Supervisor when releasing common equipment

ADDITIONAL INFORMATION

Failed Component Identification: None

Past Similar Events None

