

# CATEGORY 1

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9610040100      DOC. DATE: 96/09/27      NOTARIZED: NO      DOCKET #  
 FACIL: 50-388 Susquehanna Steam Electric Station, Unit 2, Pennsylv      05000388  
 AUTH. NAME      AUTHOR AFFILIATION  
 KICHLINE, R.D.      Pennsylvania Power & Light Co.  
 KUCZYNSKI, G.J.      Pennsylvania Power & Light Co.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 96-006-01: on 960902, both loops of Containment Radiation Monitors were removed from service during planned Maint. TS 3.4.3.1. does not adequately address isolation of both loops. 3.0.3 LCO's were cleared. W/960927 ltr.

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**Pennsylvania Power & Light Company**

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U.S. Nuclear Regulatory Commission  
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SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 50-388/96-006-00  
PLAS - 680 FILE R41-2

Docket No. 50-388  
License No. NPF-22

Attached is Licensee Event Report 50-388/96-006-00. This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B) in that both loops of the containment radiation monitors were isolated from the containment drywell at the same time during planned maintenance on the Reactor Protection System. This required entry into Technical Specification Limiting Condition for Operation (LCO) 3.0.3. Entry into LCO 3.0.3 represents a condition prohibited by the plant's Technical Specifications.

  
G. J. Kuczynski

Plant Manager - Susquehanna SES

Attachment

cc: Mr. H. J. Miller  
Regional Administrator  
U. S. Nuclear Regulatory Commission  
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Mr. Kenneth M. Jenison  
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PDR ADDCK 05000388  
S PDR

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U7.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2						DOCKET NUMBER(2) 0 5 0 0 0 3 8 8			PAGE (3) 1 OF 0 3		
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TITLE (4)  
Both Loops of Containment Radiation Monitors Removed from Service During Planned Maintenance

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)										
0	9	0	2	9	6	9	6	0	0	6	0	0	0	0	0	0	0	0	0	0	0

OPERATING MODE (9) 1		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR Y : (Check one or more of the following) (11)										
POWER LEVEL (10) 1 0 0	20.402(b)		20.405(a)(1)(i)		20.405(a)(1)(ii)		20.405(a)(1)(iii)		20.405(a)(1)(iv)		20.405(a)(1)(v)	
	20.405(a)(2)(v)		20.405(a)(2)(vi)		20.405(a)(2)(vii)		20.405(a)(2)(viii)		20.405(a)(2)(ix)		20.405(a)(2)(x)	
	20.405(a)(2)(xi)		20.405(a)(2)(xii)		20.405(a)(2)(xiii)		20.405(a)(2)(xiv)		20.405(a)(2)(xv)		20.405(a)(2)(xvi)	
	20.405(a)(2)(xvii)		20.405(a)(2)(xviii)		20.405(a)(2)(xix)		20.405(a)(2)(xx)		20.405(a)(2)(xxi)		20.405(a)(2)(xxii)	
	20.405(a)(2)(xxiii)		20.405(a)(2)(xxiv)		20.405(a)(2)(xxv)		20.405(a)(2)(xxvi)		20.405(a)(2)(xxvii)		20.405(a)(2)(xxviii)	
	20.405(a)(2)(xxix)		20.405(a)(2)(xxx)		20.405(a)(2)(xxxi)		20.405(a)(2)(xxxii)		20.405(a)(2)(xxxiii)		20.405(a)(2)(xxxiv)	
		50.73(a)(2)(v)		50.73(a)(2)(vi)		50.73(a)(2)(vii)		50.73(a)(2)(viii)		50.73(a)(2)(ix)		
		50.73(a)(2)(x)		50.73(a)(2)(xi)		50.73(a)(2)(xii)		50.73(a)(2)(xiii)		50.73(a)(2)(xiv)		
		50.73(a)(2)(xv)		50.73(a)(2)(xvi)		50.73(a)(2)(xvii)		50.73(a)(2)(xviii)		50.73(a)(2)(xix)		
		50.73(a)(2)(xx)		50.73(a)(2)(xxi)		50.73(a)(2)(xxii)		50.73(a)(2)(xxiii)		50.73(a)(2)(xxiv)		
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		50.73(a)(2)(xxxix)		50.73(a)(2)(xl)		50.73(a)(2)(xli)		50.73(a)(2)(xlii)		50.73(a)(2)(xliii)		
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		50.73(a)(2)(xlviii)		50.73(a)(2)(xlvix)		50.73(a)(2)(xli)		50.73(a)(2)(xli)		50.73(a)(2)(xli)		
		50.73(a)(2)(xli)		50.73(a)(2)(xli)		50.73(a)(2)(xli)		50.73(a)(2)(xli)		50.73(a)(2)(xli)		

(LICENSEE CONTACT FOR THIS LER (12))

NAME Robert D. Kichline - Project Licensing Specialist						TELEPHONE NUMBER AREA CODE 6 1 0 7 7 4 - 7 7 0 5					
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0210, 1219 and 1655 hours on September 2, 1996, with Unit 2 in Condition 1 (Power Operation) at 100% power, Technical Specification 3.0.3 was entered since both Containment Radiation Monitors (CRM) were removed from service at the same time during planned maintenance on the Reactor Protection System (RPS). Technical Specification 3.4.3.1 requires at least one CRM to be aligned to the containment drywell. With no CRMs aligned to the containment drywell, Technical Specification 3.0.3 has to be entered. This event is reportable per 10CFR50.73(a)(2)(i)(B) in that entry into Technical Specification LCO 3.0.3 constitutes a condition prohibited by the Technical Specifications. The CRMs are used for detection of leakage into the primary containment during normal operation and are not used post-accident. There was no safety consequence or compromise to the public health and safety as a result of both loops of the CRMs being isolated from the containment drywell at the same time. A request to revise Technical Specification 3.4.3.1 has been submitted to the NRC to address the isolation of both loops of CRMs in order to eliminate the entry into Technical Specification 3.0.3.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						PAGE (3)		
Unit 2		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER						
Susquehanna Steam Electric Station	0 5 0 0 0 3 8 8	9 6	0 0 6	0 0			2	OF	3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

**EVENT DESCRIPTION**

At 0210, 1219 and 1655 hours on September 2, 1996, with Unit 2 in Condition 1 (Power Operation) at 100% power, Technical Specification 3.0.3 was entered when both Containment Radiation Monitors (CRM; EISS Code IL) were removed from service at the same time during planned maintenance on the Reactor Protection System (RPS; EISS Code: JC). Removal of the CRMs from service was required when the power supply for the "A" RPS bus was transferred from the Normal supply to the Alternate supply and then back to the Normal supply. The design of the RPS is such that when transferring power supplies, the affected RPS bus is momentarily de-energized during the transfer. This de-energization of the RPS bus causes the system to actuate. The design of the isolation feature for the CRMs is that for an actuation of either division of RPS, both loops of the CRMs will isolate. This design feature of isolating both loops on the actuation of either RPS division meets the applicable regulatory requirements and was installed in both units in 1995. This event is reportable per 10CFR50.73(a)(2)(i)(B) in that entry into LCO 3.0.3 constitutes a condition prohibited by the Technical Specifications. Technical Specification 3.4.3.1 requires at least one CRM to be aligned to the containment drywell. With no CRMs aligned to the containment drywell, Technical Specification 3.0.3 was entered. There was no safety consequence or compromise to the public health and safety as a result of both loops of the CRMs being isolated from the containment drywell at the same time. The 3.0.3 LCO's were cleared at 0220, 1245 and 1705 on 9/2/96.

**CAUSE OF EVENT**

Technical Specification 3.4.3.1 does not adequately address the isolation of both loops of CRMs from the containment. Entry into Technical Specification 3.0.3 was therefore necessary since both loops of CRMs were isolated from the containment drywell during planned maintenance on the "A" RPS system, per RPS design.

**REPORTABILITY/ANALYSIS**

This event was determined to be reportable per 10CFR50.73(a)(2)(i)(B), in that entry into LCO 3.0.3 constitutes a condition prohibited by the Technical Specifications when both loops of the Containment Radiation Monitors were isolated from the containment drywell at the same time. Technical Specification 3.4.3.1 requires that one loop of CRMs (one gaseous channel and one particulate channel) be operable and aligned to the containment drywell during Operating Conditions 1, 2 and 3.

**LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION**

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)					PAGE (3)		
Unit 2		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER					
Susquehanna Steam Electric Station	0 5 0 0 0 3 8 8	9 6	0 0 6	0 0			3	OF	3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The isolation of both loops of CRMs does not affect the ability of the plant to shutdown safely, and the health and safety of the public were never challenged. Other leak detection systems were operable when both CRM loops were isolated; therefore, a leak into the containment drywell would have been detected. The plant response to a postulated transient was not changed as a result of this event.

In accordance with the guidelines provided in NUREG-1022, Supplement 1, Item 14.1, the required submission date for this report was determined to be October 2, 1996.

**CORRECTIVE ACTIONS**

A request to revise Technical Specification 3.4.3.1 has been submitted to the NRC to address the isolation of both loops of CRMs, and to eliminate the need for entry into Technical Specification LCO 3.0.3.

**ADDITIONAL INFORMATION**

**Past Similar Events:**

- Docket No. 50-387 LER 95-010-00
- LER 95-012-00
- LER 96-003-00
- LER 96-005-00

Failed Component: None